

Étude préliminaire des conséquences d'un arrêt des pompes

Manuele Aufiero - LPSC/CNRS & Olivier Geoffroy - INOPRO

Simulation performed in the framework of a collaboration between POLIMI and INOPRO within the EVOL Project



NUCLEAR REACTORS GROUP



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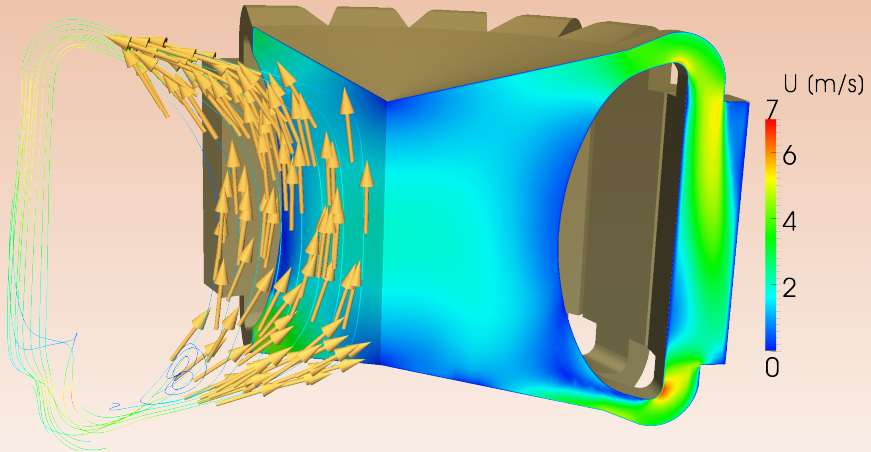
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Simulation tools have been developed within the EVOL project for detailed coupled CFD/Neutronic analysis on “real” 3D full-core MSFR geometries.

- Verification, “code-to-code” assessment
(see presentations of tomorrow)
- Study of possible accidental scenarios on the optimized MSFR geometry



OpenFOAM solver: coupling neutronics and CFD



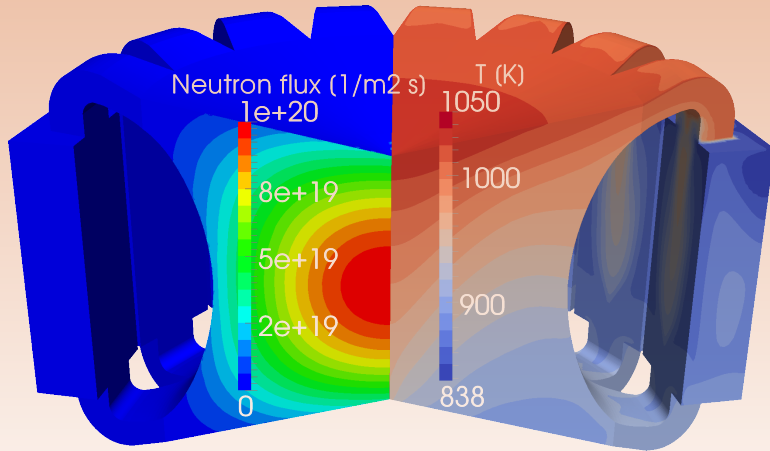
Coupled CFD/neutronics calculation for the MSFR: velocity field



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OpenFOAM solver: coupling neutronics and CFD



Coupled CFD/neutronics calculation for the MSFR: ϕ & T



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Accidental scenarios – examples



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Reactivity insertion – accidental Uranium release.



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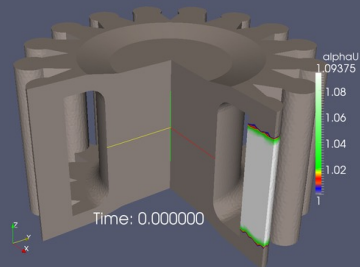
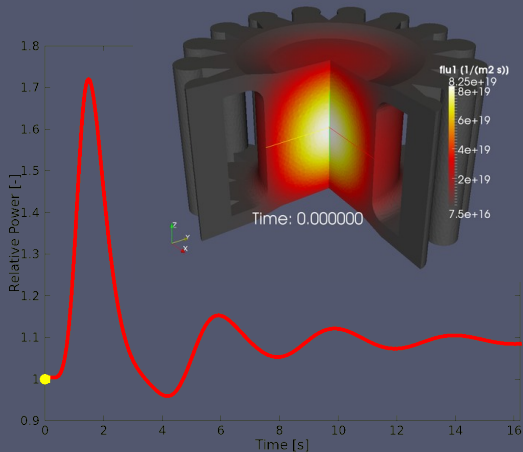
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Loss of flow.



Accidental Uranium release

Accidental release of ~ 15 kg of ^{233}U in one of the external loops

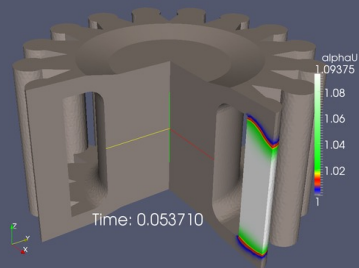
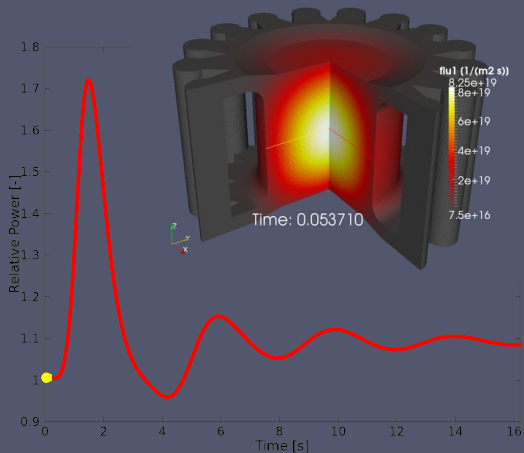


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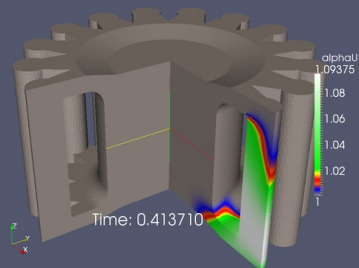
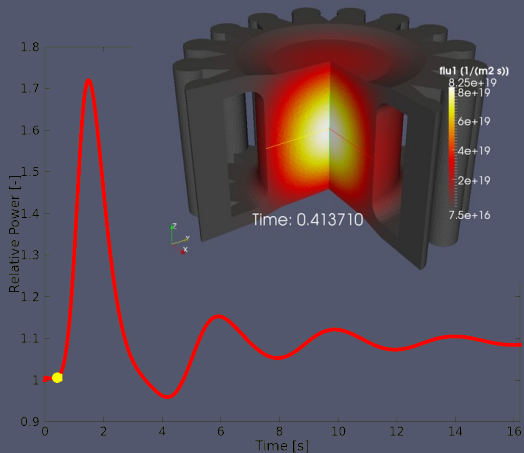


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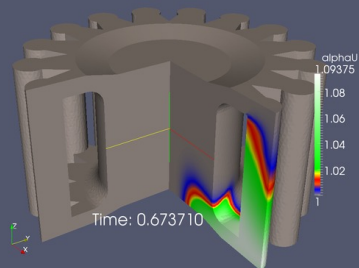
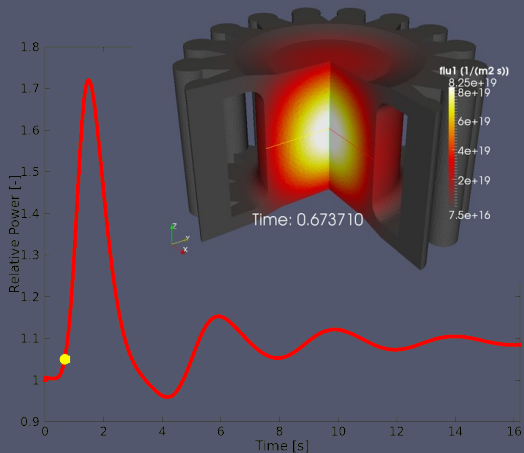


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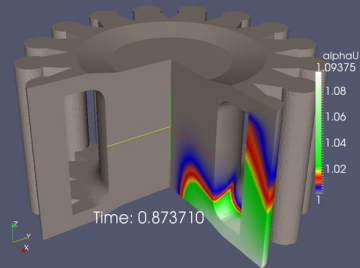
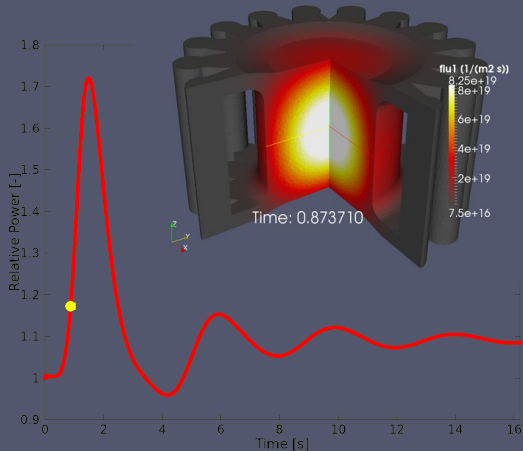


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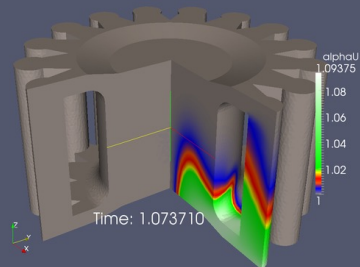
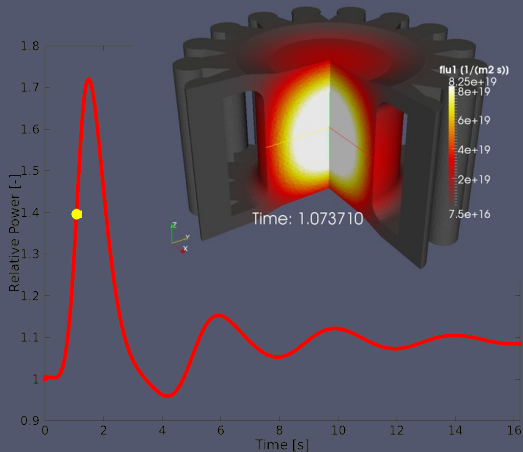


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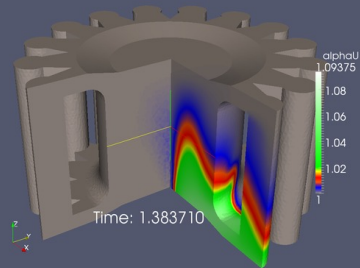
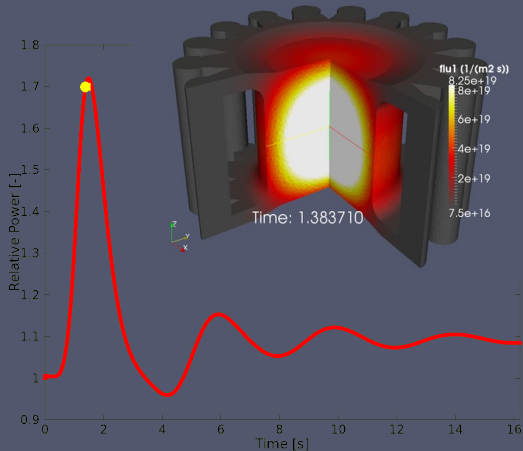


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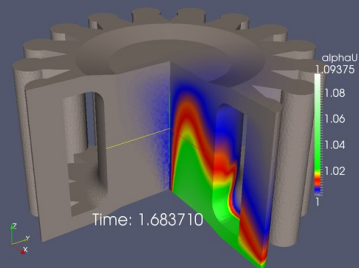
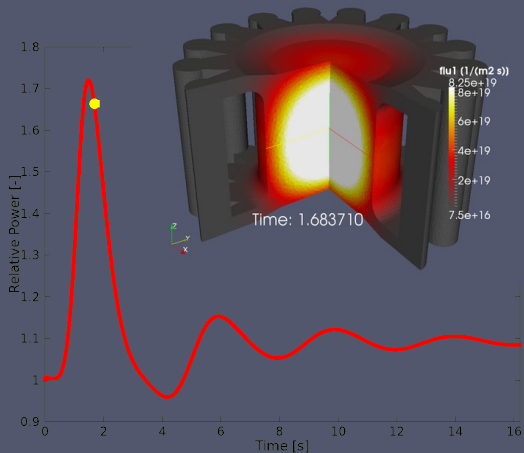


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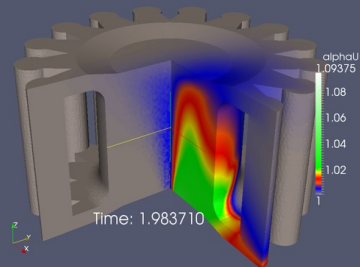
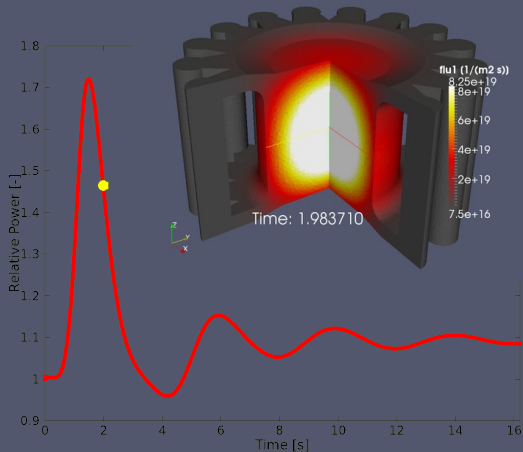


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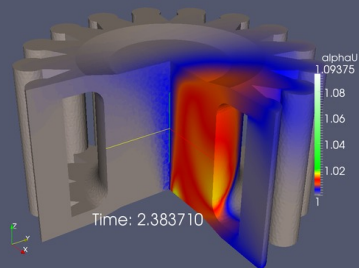
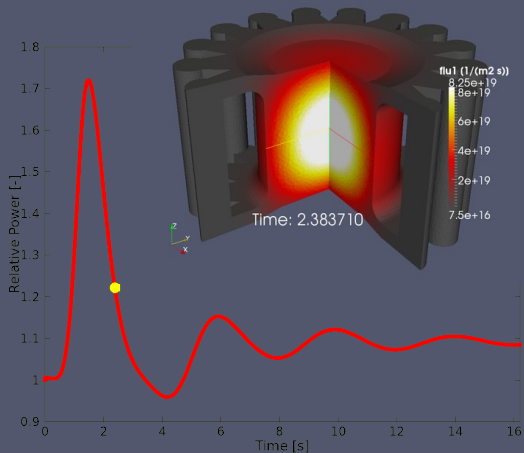


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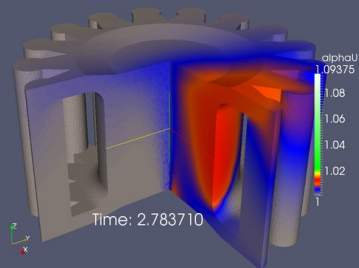
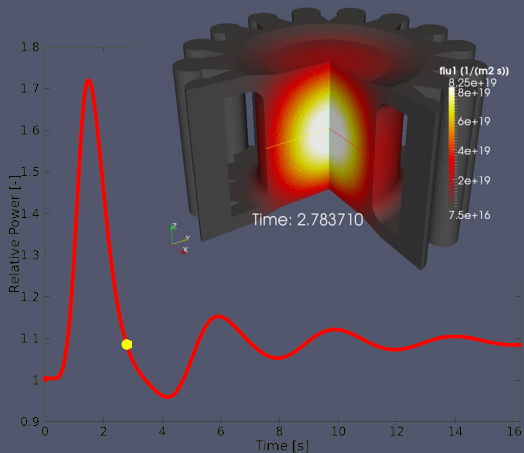


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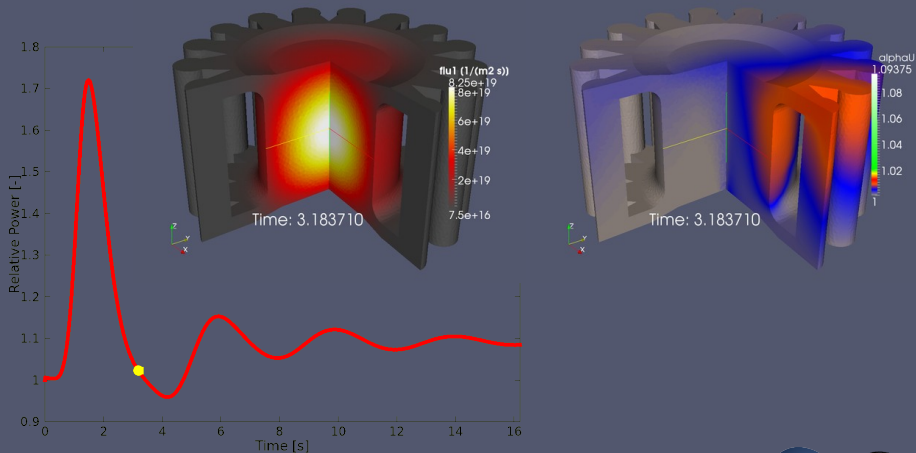


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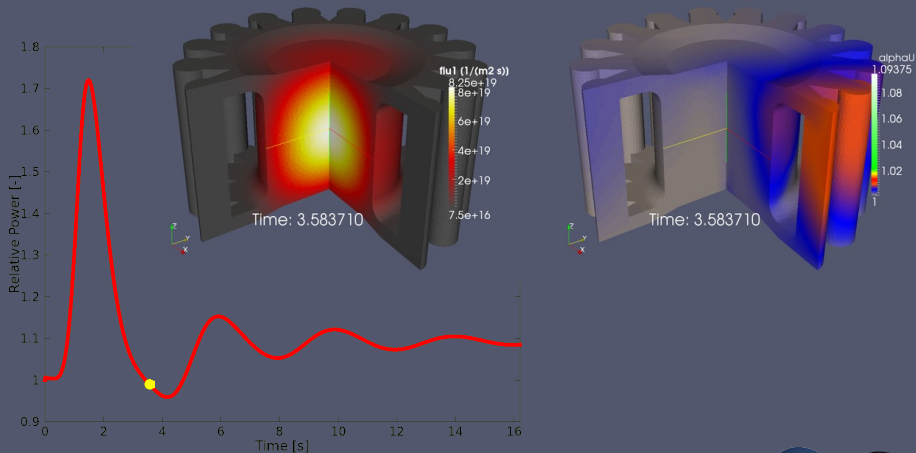


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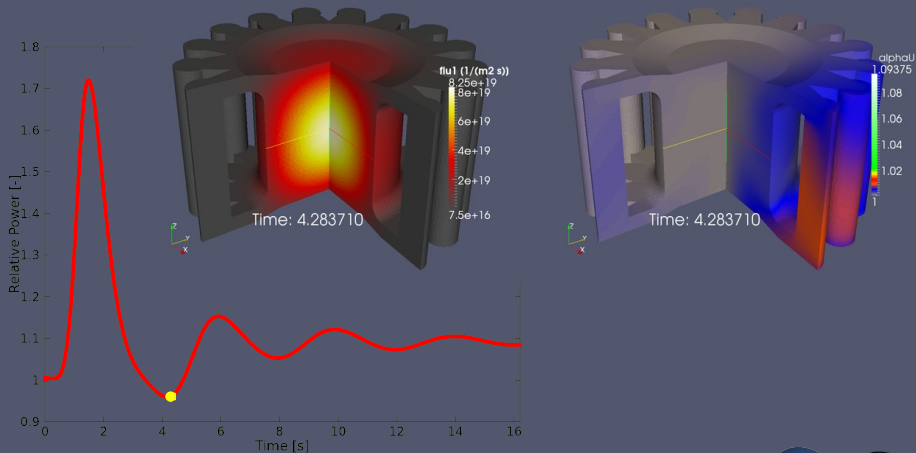


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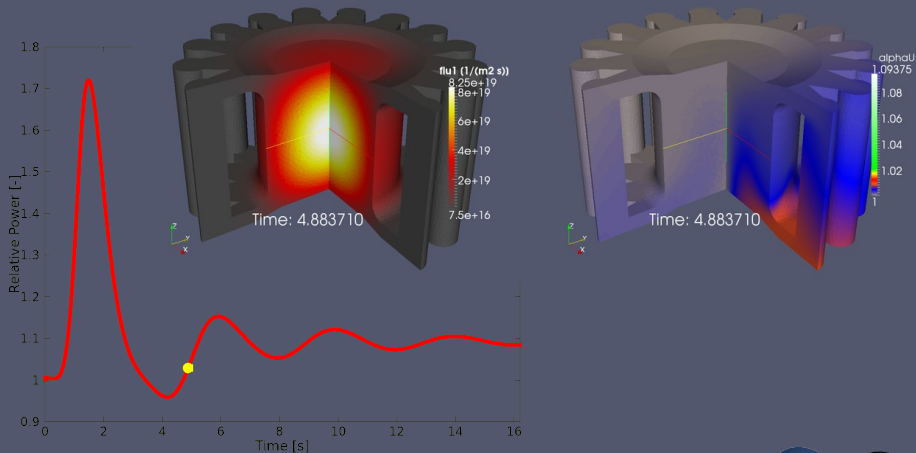


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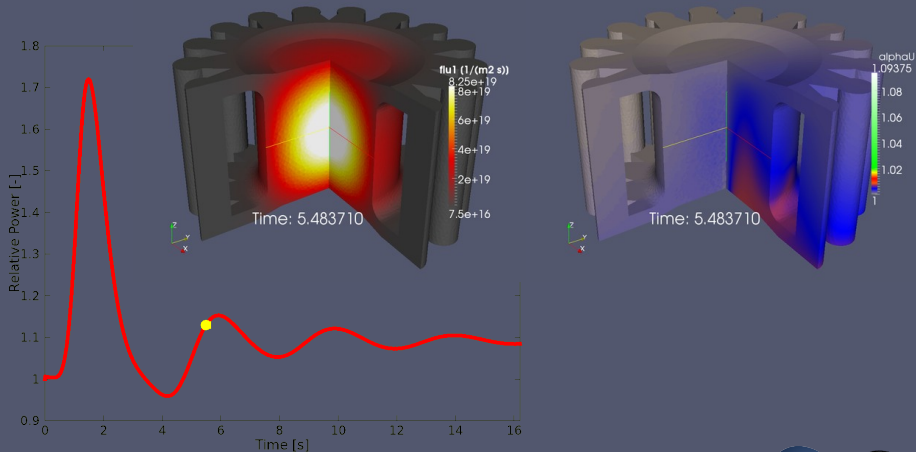


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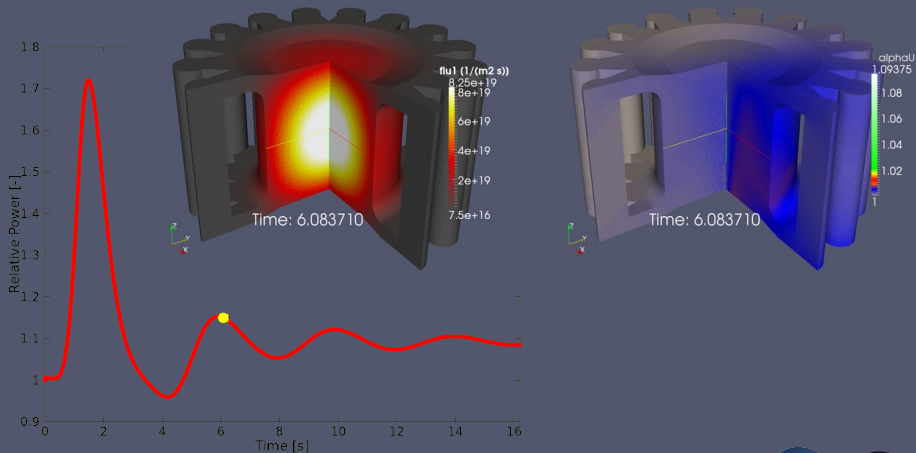


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Accidental scenarios – examples

Reactivity insertion – accidental Uranium release.

Single pump failure.

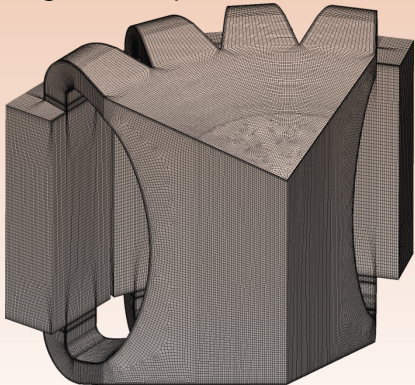
Loss of flow.



Single pump failure (one out of 16)

Accidental scenario

Unprotected pump blockage at $t = 0$ (e.g., bearing failure) after long reactor operation

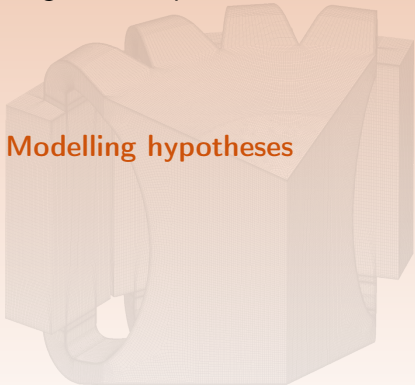


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Modelling hypotheses



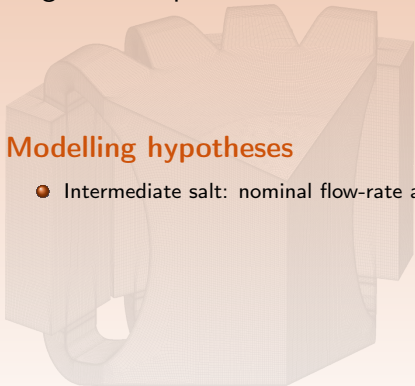
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- Intermediate salt: nominal flow-rate and inlet temperature (**500°C**)



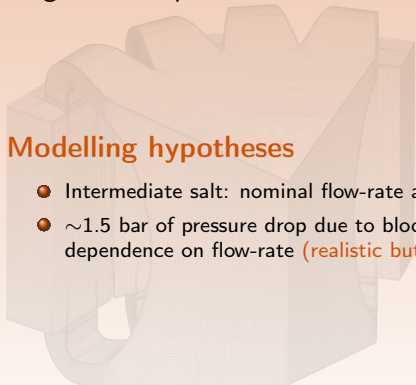
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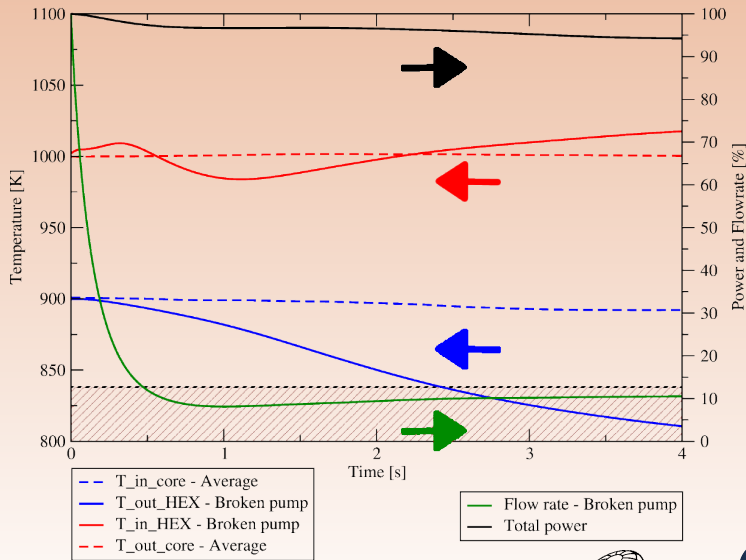
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- Cross-sections, group constants and decay heat curves relative to equilibrium fuel salt composition (**anyway, results are not too sensitive**)



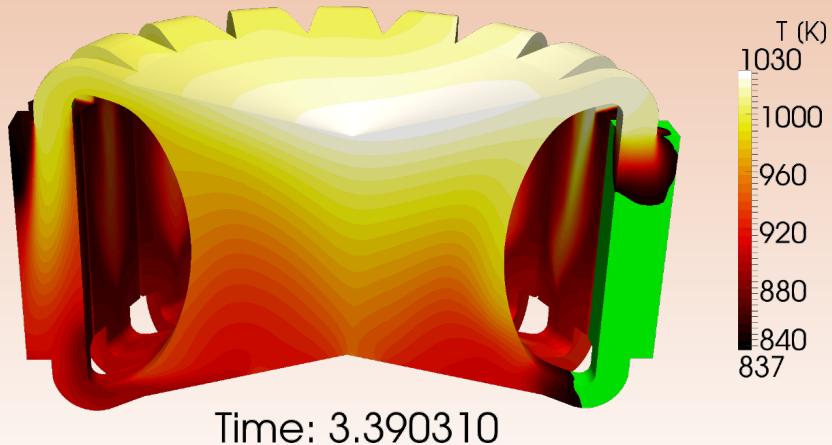
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Single pump failure – What happens?

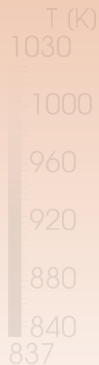
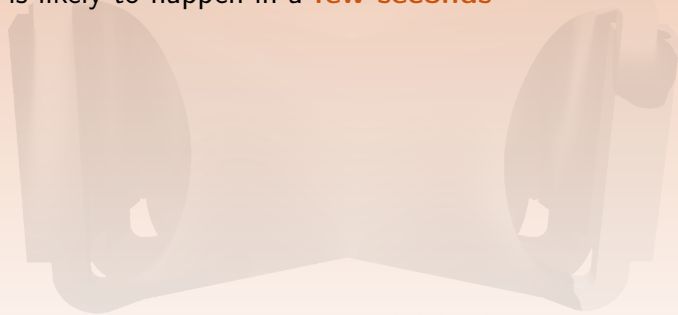


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Despite the **heavy simplifications** of the simulation, partial (or total!) **salt solidification** in the HEX is likely to happen in a **few seconds**



Time: 3.390310



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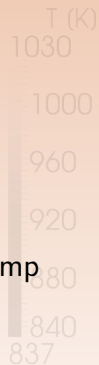


Single pump failure – What happens?

Despite the **heavy simplifications** of the simulation, partial (or total!) **salt solidification** in the HEX is likely to happen in a **few seconds**

Stagnant fuel salt above the HEX and in the broken pump is quickly heated up by **decay heat**

Time: 3.390310



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Accidental scenarios – examples

Reactivity insertion – accidental Uranium release.

Single pump failure.

Loss of flow.



Accidental scenario

Loss of power of all the pump motors

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Modelling hypotheses

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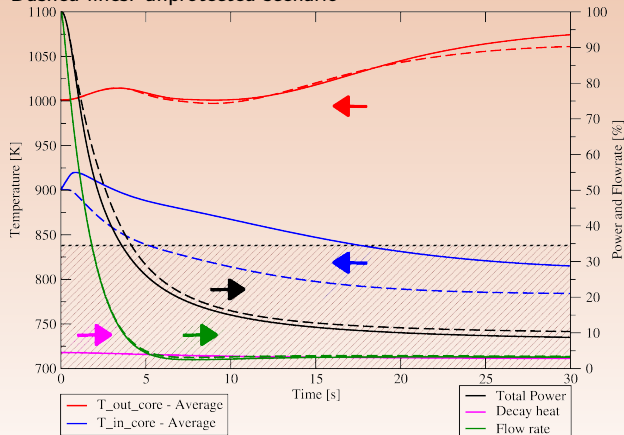
- Exponential pump coast down, $\tau = 1$ or 15 s (small to medium inertial flywheel)
- Nominal intermediate salt inlet temperature (500°C)
- Unprotected scenario: nominal intermediate salt flow rate
"Mitigated" scenario: fast reduction of intermediate salt flow rate to 10 %

Loss of flow – What happens?

Low flywheel inertia: $\tau = 1$ s

Solid lines: fast reduction of intermediate salt flow rate

Dashed lines: unprotected scenario



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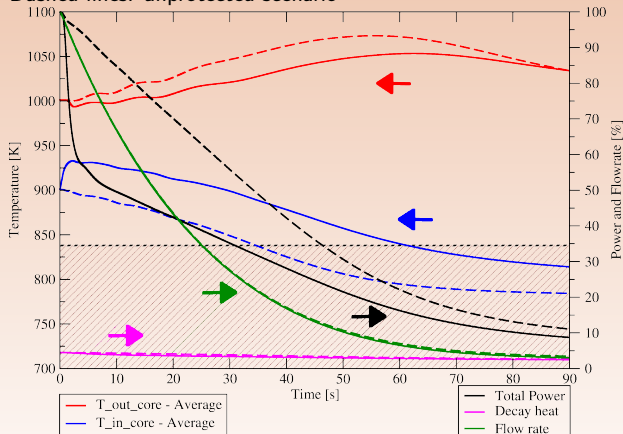


Loss of flow – What happens?

Medium flywheel inertia: $\tau = 15$ s

Solid lines: fast reduction of intermediate salt flow rate

Dashed lines: unprotected scenario

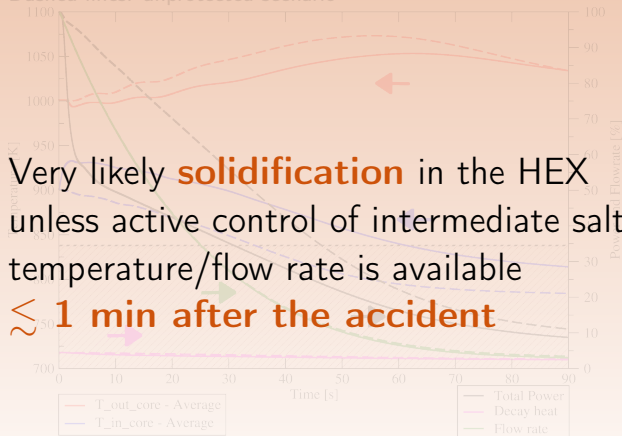


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Mitigation of pump failure – some possible solution

- prevent salt solidification



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- prevent salt solidification
- limit salt solidification



Mitigation of pump failure – some possible solution

- prevent salt solidification
- limit salt solidification
- endure salt solidification



Mitigation of pump failure – some possible solution

- prevent salt solidification

- Increase of intermediate salt temperature

Intermediate salt inlet temperature: $\gtrsim 565^{\circ}\text{C}$

- limit salt solidification

- + Solidification is avoided “by design” until intermediate salt temperature is kept above freezing point

- endure salt solidification

- Higher HEX volume is required

- ? Problems at steam generator side(?)



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Mitigation of pump failure – some possible solution

- prevent salt solidification
 - Increase of intermediate salt temperature
 - 2 pumps – 1 HEX module
- limit salt solidification
- endure salt solidification

Coupling 2 pumps (in parallel) for a single HEX

- + Solidification is avoided if enough flow rate is ensured (i.e. $\gtrsim$ 20-30%)
- Ineffective against common cause failure



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Mitigation of pump failure – some possible solution

- prevent salt solidification

- Increase of intermediate salt temperature
- 2 pumps – 1 HEX module

- limit salt solidification

- Ad hoc HEX design

- endure salt solidification

E.g., tube within tube

Frozen salt at tube walls acts as thermal barrier and prevents further solidification

- + Molten passage area is ensured INSIDE the HEX
- Higher HEX volume is required
- ? Never tested(?)



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Mitigation of pump failure – some possible solution

- prevent salt solidification
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 - Large “by-pass” area
- endure salt solidification

A significant fraction of the fuel stream flows in a “by-pass” area

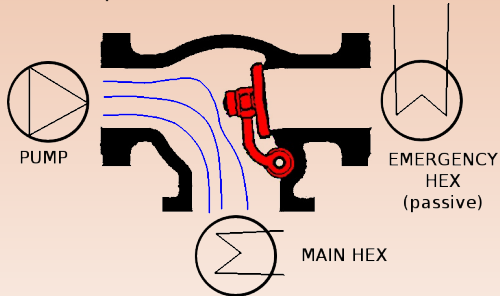
- + Molten passage area is ensured OUTSIDE the HEX
- Loss in efficiency



Mitigation of pump failure – some possible solution

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 - Passively actuated valve & HEX

Nominal operation



In nominal operation “by-pass” is closed by pressure difference



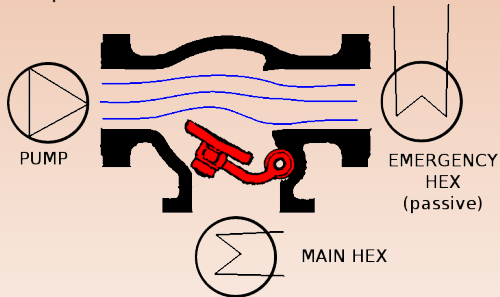
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Pump failure

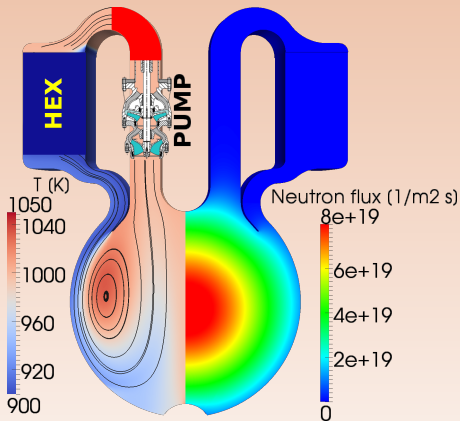
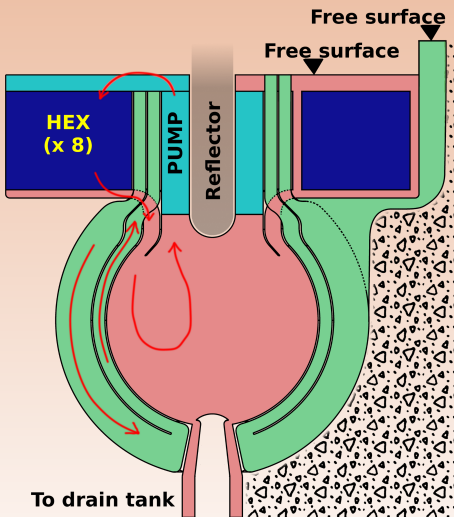


After pump failure, “by-pass” towards emergency HEX opens



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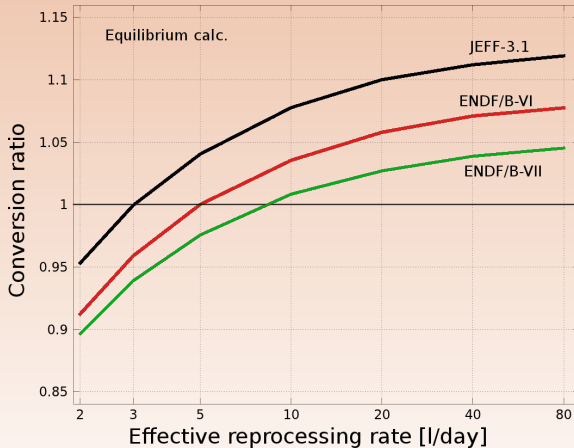


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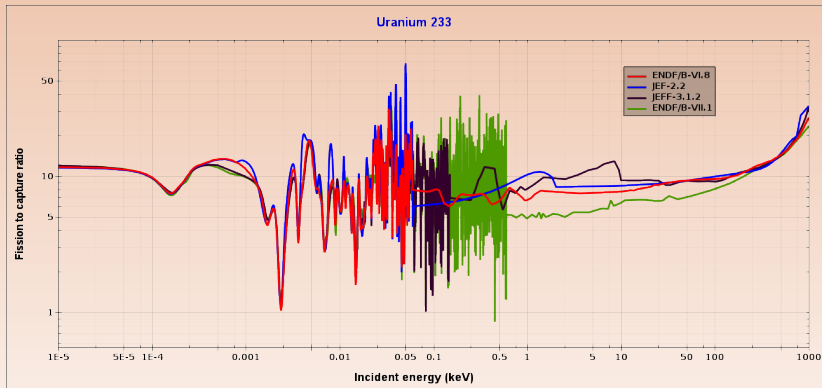


JEFF vs ENDF: equilibrium CR

From JEFF-3.1 to ENDF/B-VII, break-even reprocessing rate prediction doubles.



^{233}U Fission to capture ratio



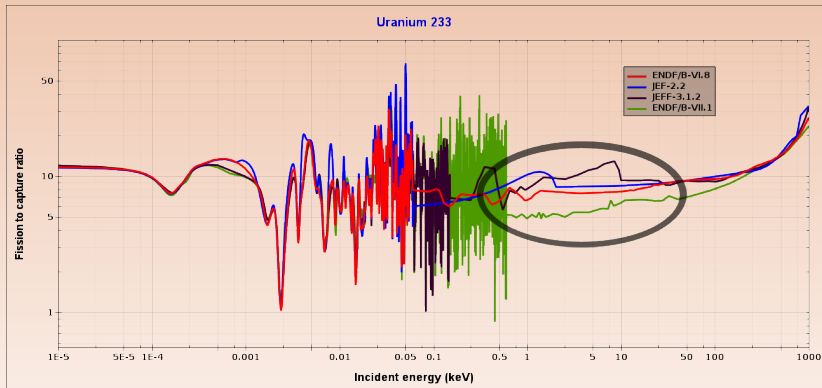
Good agreement between the libraries almost everywhere...



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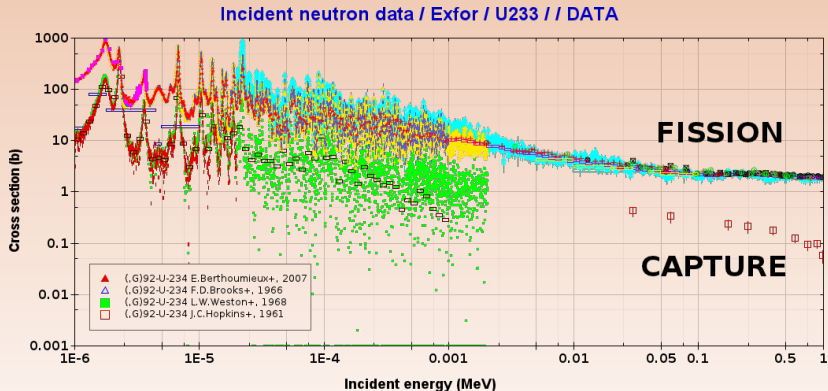
^{233}U Fission to capture ratio



...NOT in the MSFR energy spectrum region.

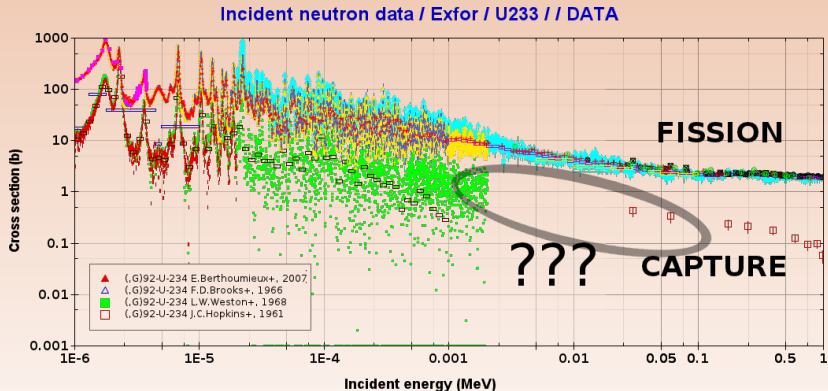
^{233}U Fission to capture ratio

Simple research on the EXFOR database...



^{233}U Fission to capture ratio

Simple research on the EXFOR database... something missing?



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THANK YOU FOR THE ATTENTION



Vue sur l'agglomération Grenobloise depuis le sommet du Moucherotte (Bertrand93)

QUESTIONS? SUGGESTIONS? NEW IDEAS?