

Presentation of the MSFR reactor concept

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Concept of Molten Salt Fast Reactor (MSFR)

Advantages of a Liquid Fuel

- ✓ Homogeneity of the fuel (no loading plan)
- ✓ Fuel = coolant \Rightarrow Heat produced directly in the heat transfer fluid
- ✓ Possibility to reconfigure quickly and passively the geometry of the fuel (gravitational draining)
- ✓ Possibility to reprocess the fuel without stopping the reactor

+ Gen4 criteria \Rightarrow step1 = Neutronic optimization of MSR:

- **Safety:** negative feedback coefficients
- **Sustainability:** reduce irradiation damages in the core
- **Deployment:** good breeding of the fuel + reduced initial fissile inventory



2008: Definition of an innovative MSR concept based on a fast neutron spectrum, and called MSFR (Molten Salt Fast Reactor)

- All feedback reactivity coefficients negative
- No solid material in the high flux area: reduction of the waste production of irradiated structural elements and less in core maintenance operations
- Good breeding of the fissile matter thanks to the fast neutron spectrum
- Actinides burning improved thanks to the fast neutron spectrum

R&D objectives

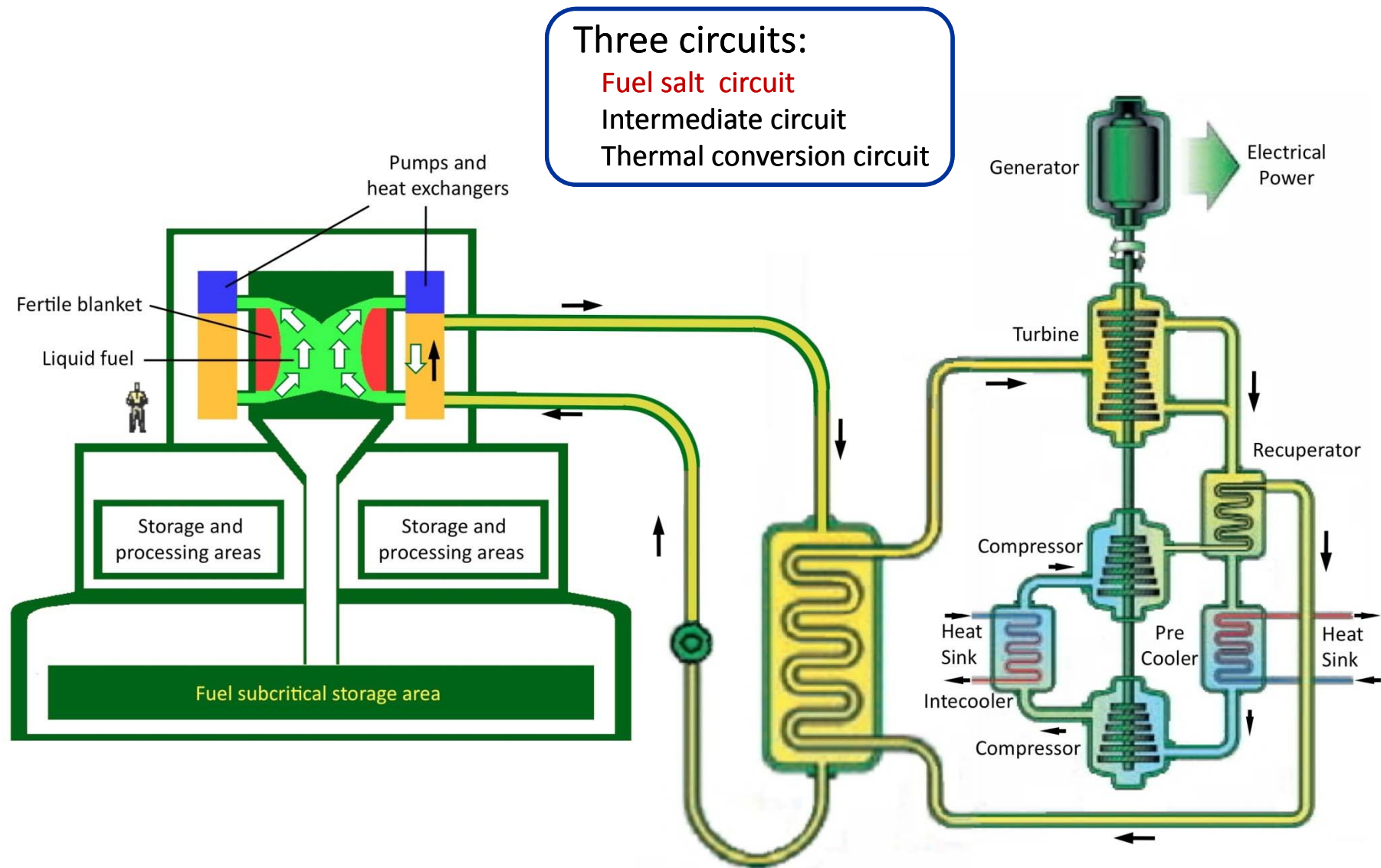
The renewal and diversification of interests in molten salts have led the MSR provisional SSC to shift the R&D orientations and objectives initially promoted in the original Generation IV Roadmap issued in 2002, in order to encompass in a consistent body the different applications envisioned today for fuel and coolant salts.

Two baseline concepts are considered which have large commonalities in basic R&D areas, particularly for liquid salt technology and materials behavior (mechanical integrity, corrosion):

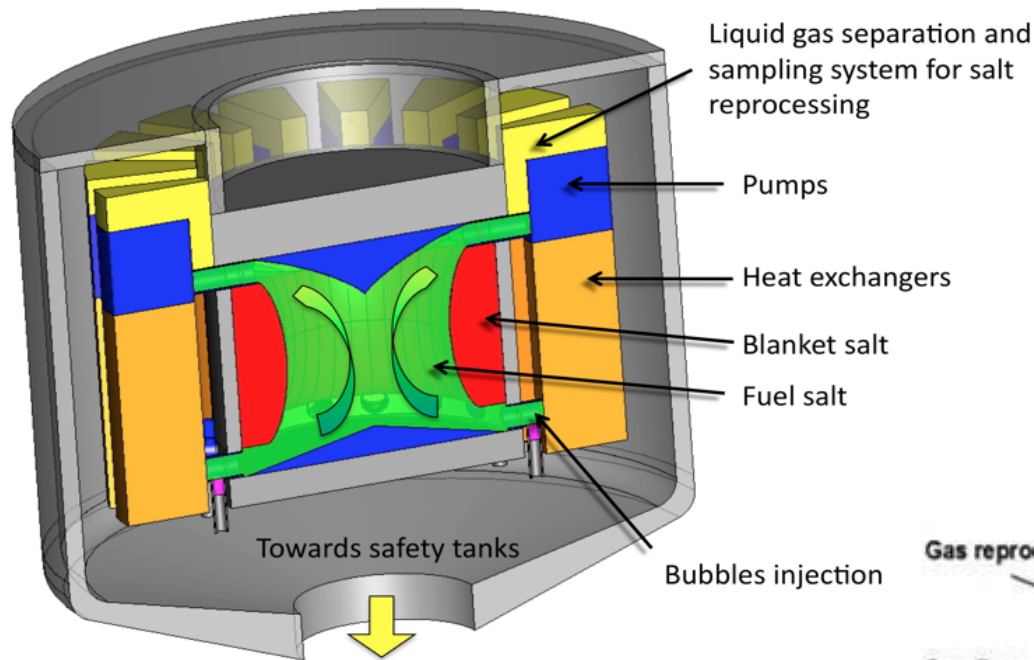
- The Molten Salt Fast-neutron Reactor (MSFR) is a long-term alternative to solid-fuelled fast neutron reactors offering very negative feedback coefficients and simplified fuel cycle. Its potential has been assessed but specific technological challenges must be addressed and the safety approach has to be established.

- The AHTR is a high temperature reactor with better compactness than the VHTR and passive safety potential for medium to very high unit power.

Molten Salt Fast Reactor (MSFR)



Molten Salt Fast Reactor (MSFR): fuel circuit



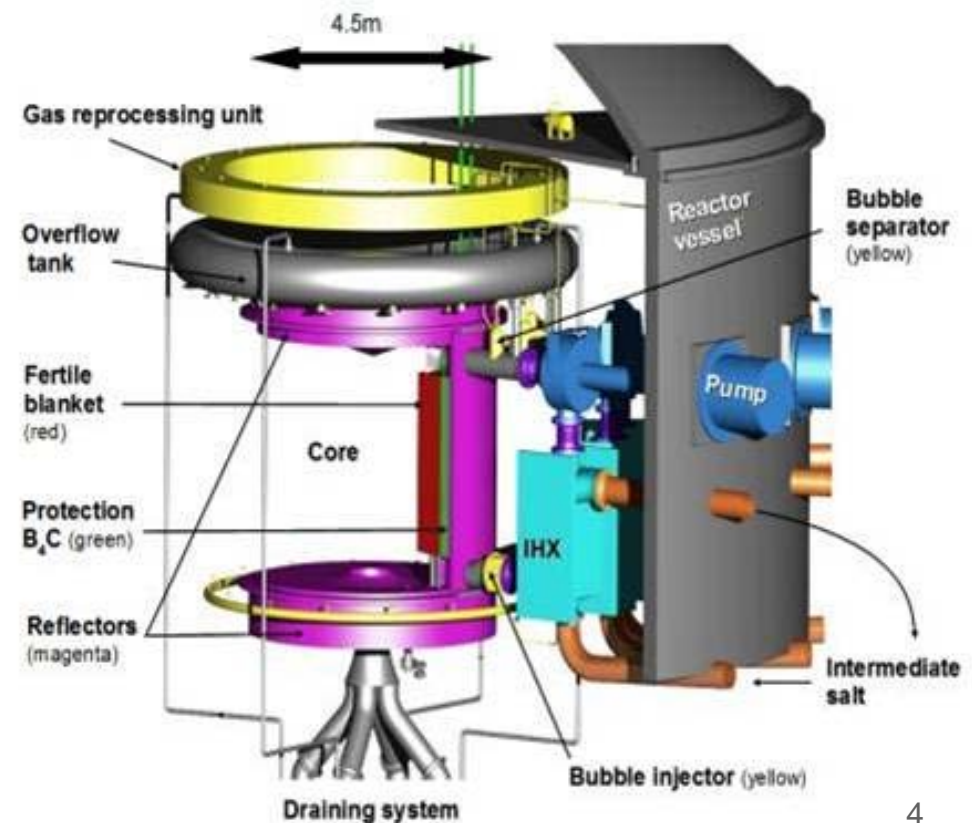
Core (active area):

No inside structure

Outside structure: Upper and lower Reflectors, Fertile Blanket Wall

+ 16 external recirculation loops:

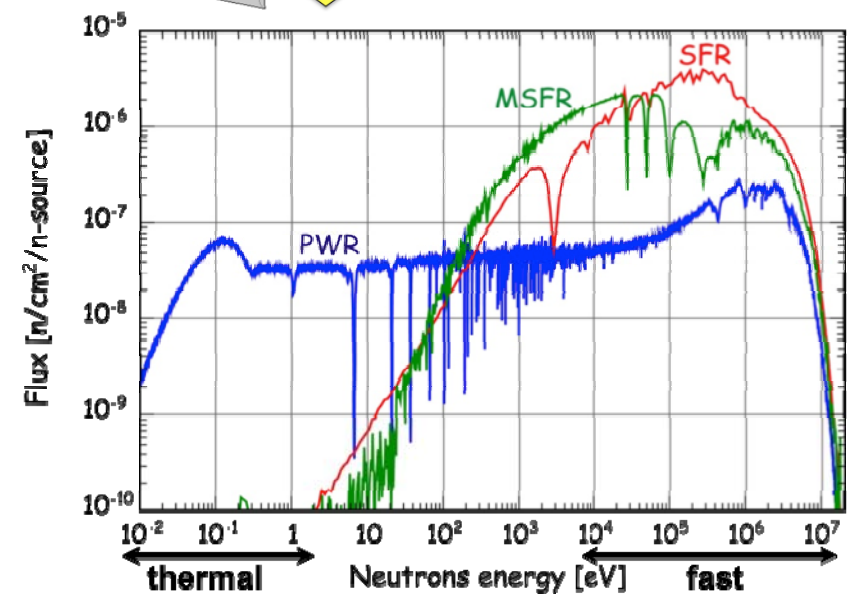
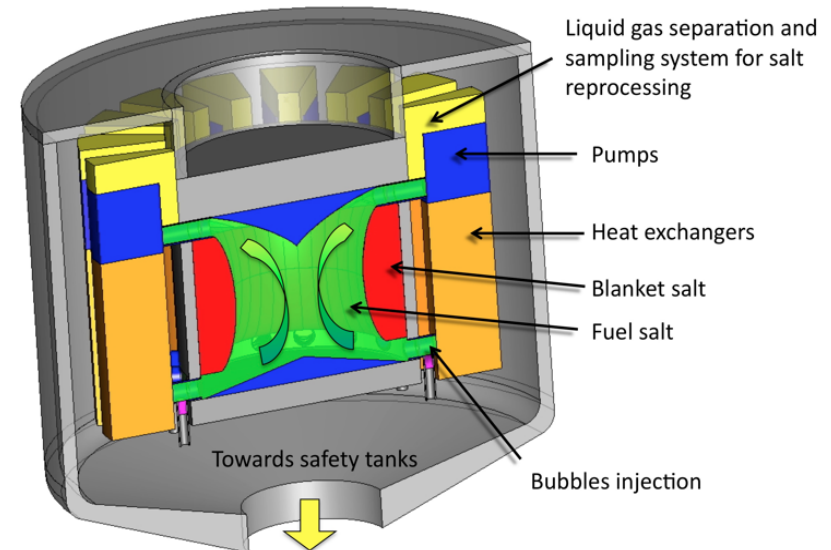
- Pipes (cold and hot region)
- Bubble Separator
- Pump
- Heat Exchanger
- Bubble Injection



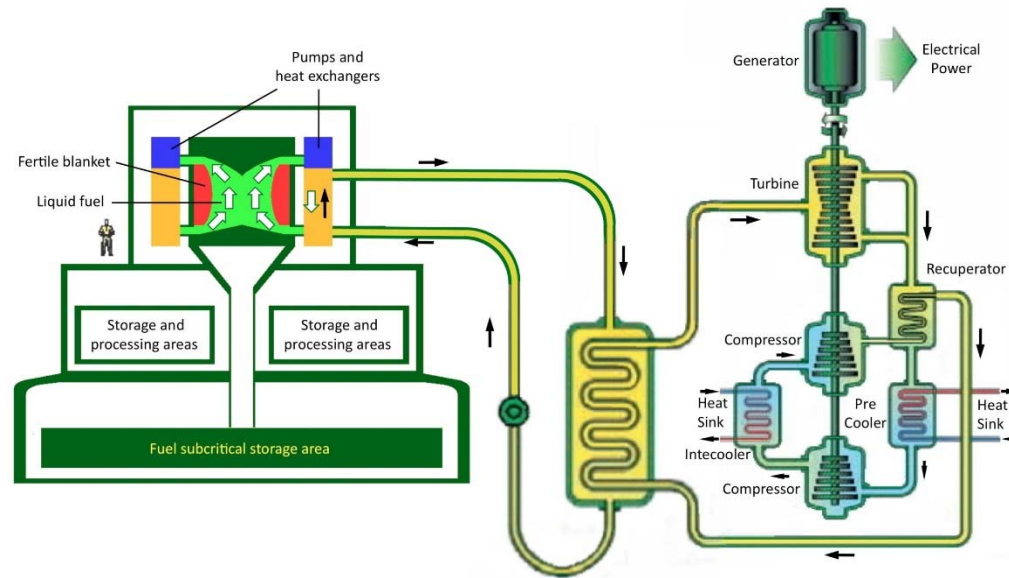
The concept of Molten Salt Fast Reactor (MSFR)

Thermal power	3000 MWth
Mean fuel salt temperature	750 °C
Fuel salt temperature rise in the core	100 °C
Fuel molten salt - Initial composition	77.5% LiF and 22.5% [ThF ₄ + (Fissile Matter)F ₄] with Fissile Matter = ²³³ U / enriched U / Pu+MA
Fuel salt melting point	565 °C
Fuel salt density	4.1 g/cm ³
Fuel salt dilation coefficient	8.82 10 ⁻⁴ / °C
Fertile blanket salt - Initial composition	LiF-ThF ₄ (77.5%-22.5%)
Breeding ratio (steady-state)	1.1
Total feedback coefficient	-5 pcm/K
Core dimensions	Diameter: 2.26 m Height: 2.26 m
Fuel salt volume	18 m ³ (½ in the core + ½ in the external circuits)
Blanket salt volume	7.3 m ³
Total fuel salt cycle	3.9 s

Design of the 'reference' MSFR



Concept of Molten Salt Fast Reactor (MSFR)



Next step: requires multidisciplinary expertise (reactor physics, simulation, chemistry, safety, materials, design...) from academic and industrial worlds



Cooperation frames:

- **Worldwide:** Generation 4 International Forum (GIF)
- **European:** collaborative project Euratom/Rosatom EVOL (FP7) – European project SAMOFAR (H2020) + SNETP SRIA Annex
- **National:** IN2P3/CNRS and interdisciplinary programs PACEN and NEEDS (CNRS, CEA, IRSN, AREVA, EdF), structuring project 'CLEF' of Grenoble Institute of Technology

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MSFR and the European project EVOL

European Project “EVOL” Evaluation and Viability Of Liquid fuel fast reactor FP7 (2011-2013): Euratom/Rosatom cooperation

Objective : to propose a design of MSFR by end of 2013 given the best system configuration issued from physical, chemical and material studies

- Recommendations for the design of the core and fuel heat exchangers
- Definition of a safety approach dedicated to liquid-fuel reactors - Transposition of the defence in depth principle - Development of dedicated tools for transient simulations of molten salt reactors
- Determination of the salt composition - Determination of Pu solubility in LiF-ThF₄ - Control of salt potential by introducing Th metal
- Evaluation of the reprocessing efficiency (based on experimental data) – FFER project
- Recommendations for the composition of structural materials around the core



WP2: Design and Safety

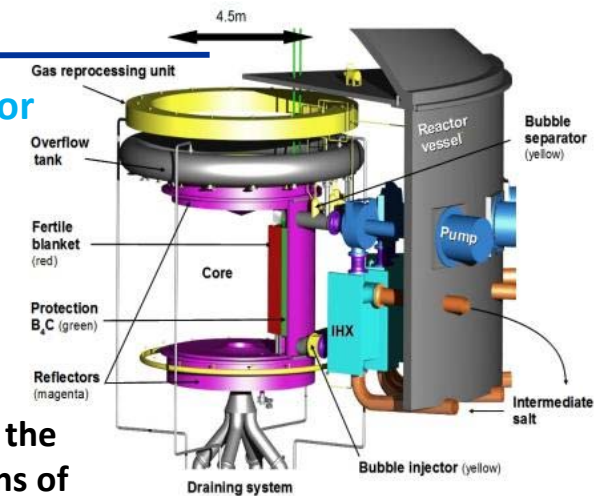
WP3: Fuel Salt Chemistry and Reprocessing

WP4: Structural Materials

12 European Partners: France (CNRS: Coordinateur, Grenoble INP , INOPRO, Aubert&Duval), Pays-Bas (Université Techno. de Delft), Allemagne (ITU, KIT-G, HZDR), Italie (Ecole polytechnique de Turin), Angleterre (Oxford), Hongrie (Univ Techno de Budapest)
+ 2 observers since 2012 : Politecnico di Milano et Paul Scherrer Institute

+ Coupled to the MARS (Minor Actinides Recycling in Molten Salt) project of ROSATOM (2011-2013)

Partners: RIAR (Dimitrovgrad), KI (Moscow), VNIITF (Snezinsk), IHTe (Ekateriburg), VNIKHT (Moscow) et MUCATEX (Moscow)

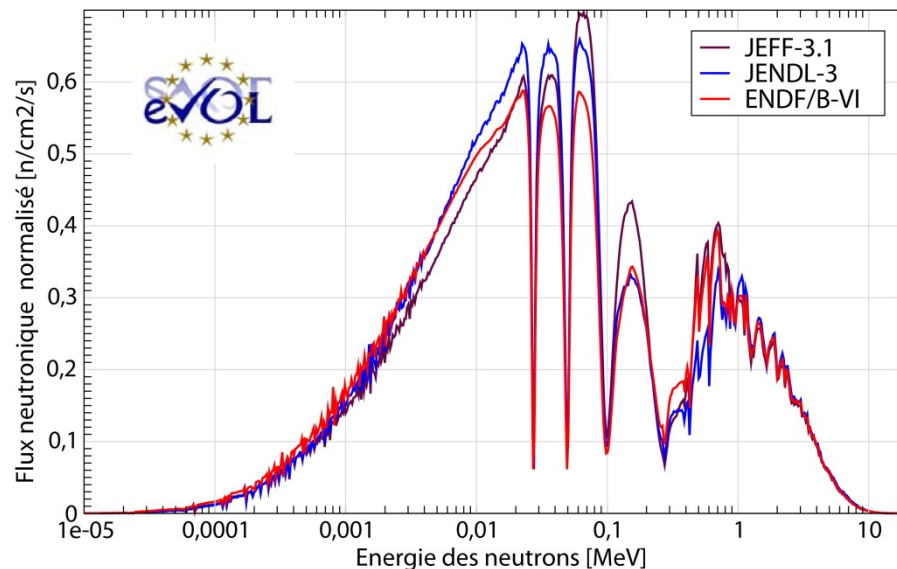


MSFR optimization: neutronic benchmark (EVOL)

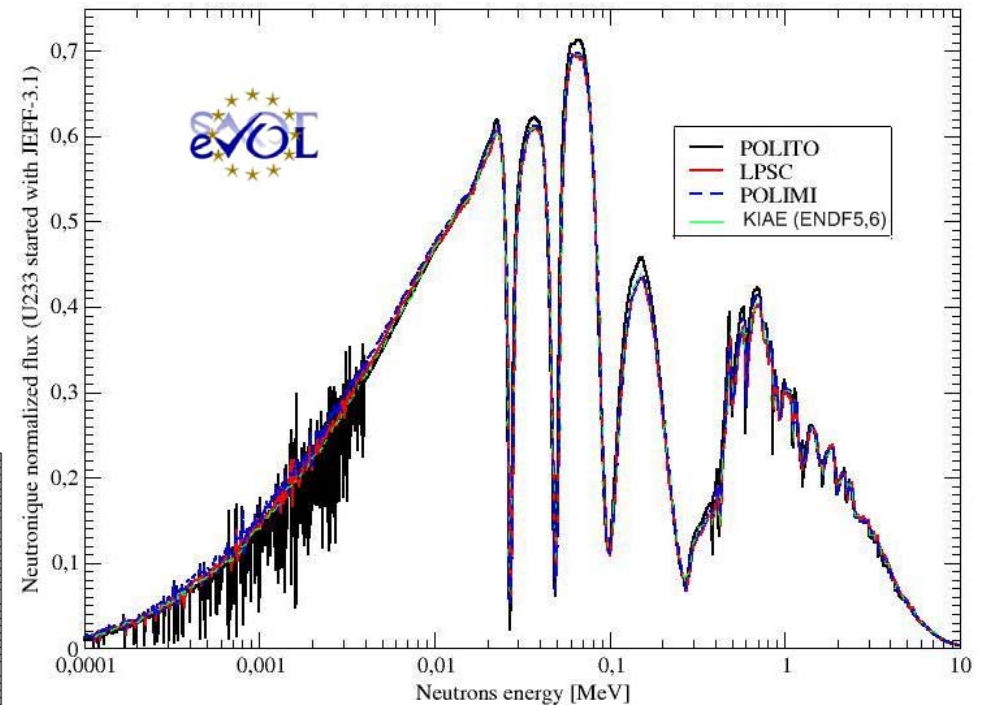
POLIMI calculations performed with SERPENT

Initial Fuel Salt Composition – EVOL Benchmark			
²³³ U-started MSFR		TRU-started MSFR	
Th	²³³ U	Th	Actinides
38 281 kg	4 838 kg	30 619 kg	Pu 11 079 kg 5.628 %mol
19.985 %mol	2.515 %mol	16.068 %mol	Np 789 kg 0.405 %mol
			Am 677 kg 0.341 %mol
			Cm 116 kg 0.058 %mol

PhD Thesis of M. Brovchenko

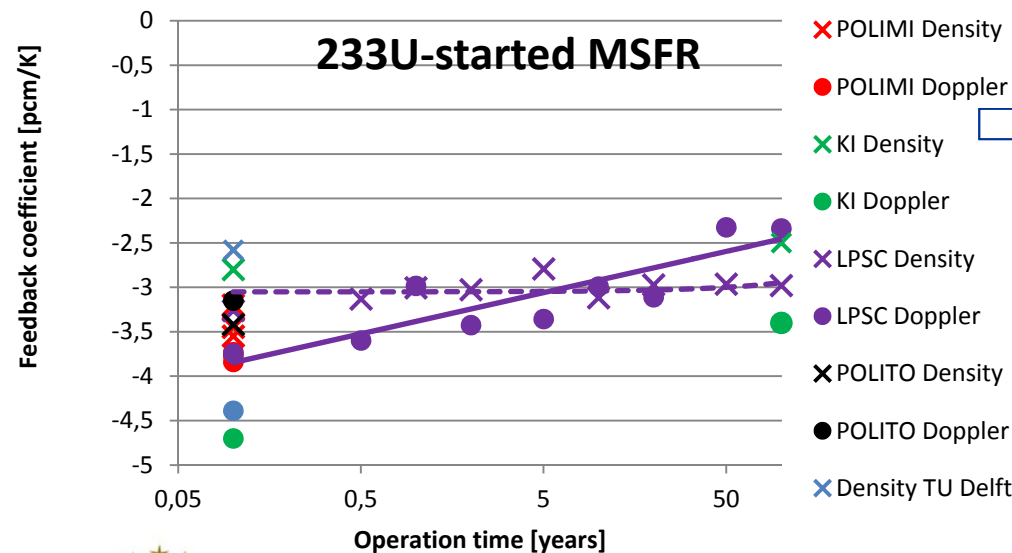


LPSC-IN2P3 calculations performed with MCNP
(coupled to in-house material evolution code REM)

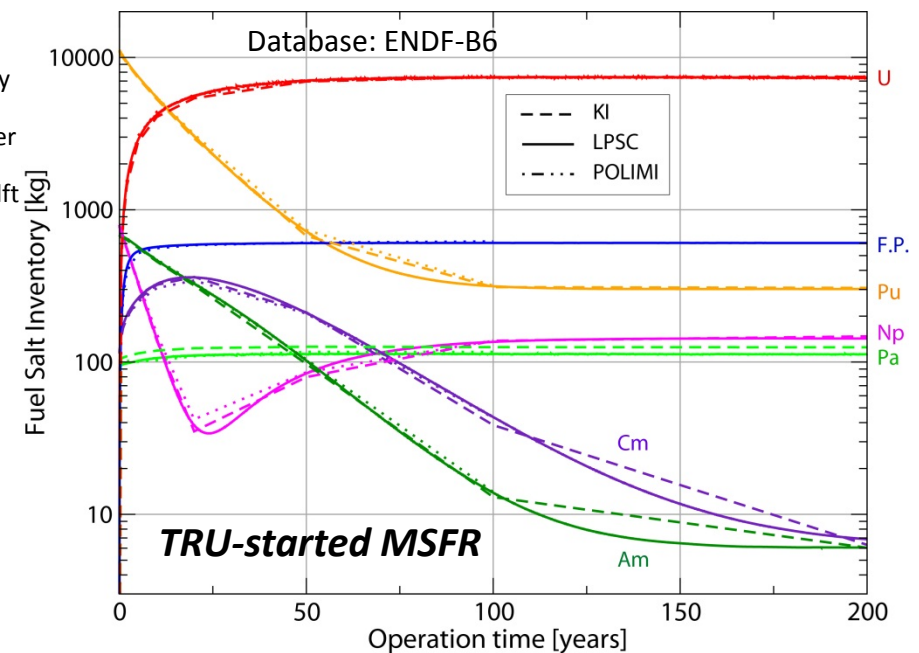
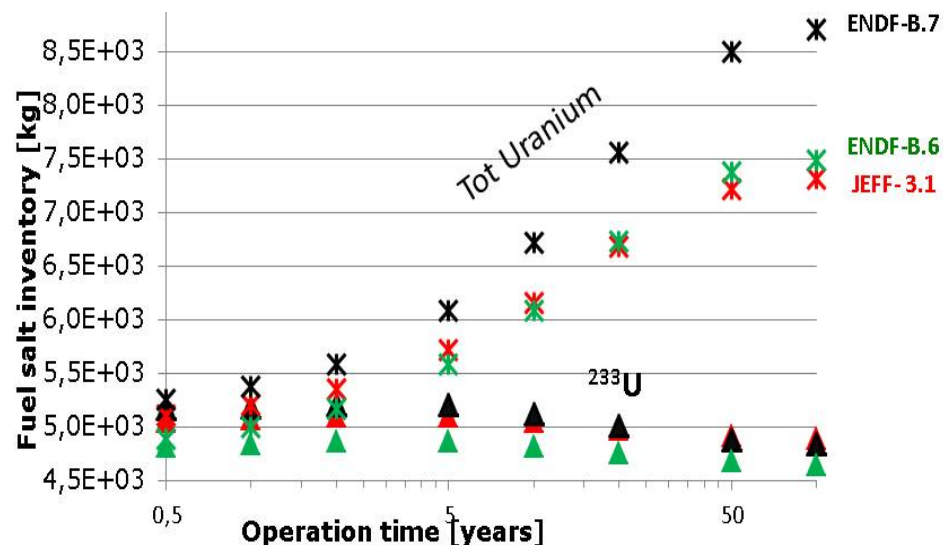


Static calculations (BOL here):
Good agreement between the different
simulation tools – High impact of the
nuclear database

MSFR optimization: neutronic benchmark (EVOL)



Largely negative feedback coefficients, \forall the simulation tool or the database used



Evolution calculations:
Very good agreement between the different simulation tools – High impact of the nuclear database

MSFR and Safety Evaluation

Design aspects impacting the MSFR safety analysis

- Liquid fuel
 - ✓ Molten fuel salt acts as reactor fuel and coolant
 - ✓ Relative uniform fuel irradiation
 - ✓ A significant part of the fissile inventory is outside the core
 - ✓ Fuel reprocessing and loading during reactor operation
- No control rods in the core
 - ✓ Reactivity is controlled by the heat transfer rate in the HX + fuel salt feedback coefficients, continuous fissile loading, and by the geometry of the fuel salt mass
 - ✓ No requirement for controlling the neutron flux shape (no DNB, uniform fuel irradiation, etc.)
- Fuel salt draining
 - ✓ Cold shutdown is obtained by draining the molten salt from the fuel circuit
 - ✓ Changing the fuel geometry allows for adequate shutdown margin and cooling
 - ✓ Fuel draining can be done passively or by operator action

MSFR and Safety Evaluation

Safety analysis: objectives

- **Develop a safety approach dedicated to MSFR**
 - **Based on current safety principles** e.g. defense-in-depth, multiple barriers, the 3 safety functions (reactivity control, fuel cooling, confinement) etc. but adapted to the MSFR.
 - Integrate both **deterministic and probabilistic** approaches
 - Specific approach dedicated to **severe accidents**:
 - Fuel liquid during normal operation
 - Fuel solubility in water (draining tanks)
 - Source term evaluation
- **Build a reactor risk analysis model**
 - Identify the **initiators and high risk scenarios** that require detailed transient analysis
 - Evaluate the risk due to the **residual heat and the radioactive inventory** in the whole system, including the reprocessing units (chemical and bubbling)
 - Evaluate some potential design solutions (**barriers**)
 - Allow reactor designer to estimate impact of design changes (***design by safety***)

H2020 SAMOFAR project – Safety Assessment of a MOlten salt FAst Reactor

« A Paradigm Shift in Nuclear Reactor Safety with the Molten Salt Fast Reactor »

(2015-2019 – Around 3 Meuros)

Partners: TU-Delft (leader), CNRS, JRC-ITU, CIRTEN (POLIMI, POLITO), IRSN, AREVA, CEA, EDF, KIT, PSI + CINVESTAV



5 technical work-packages:

WP1 Integral safety approach and system integration

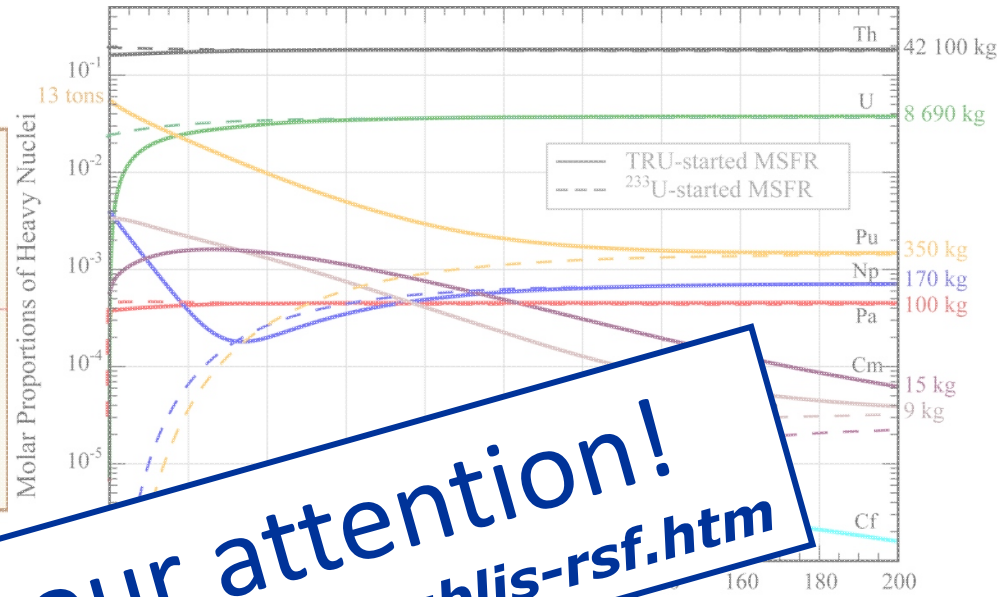
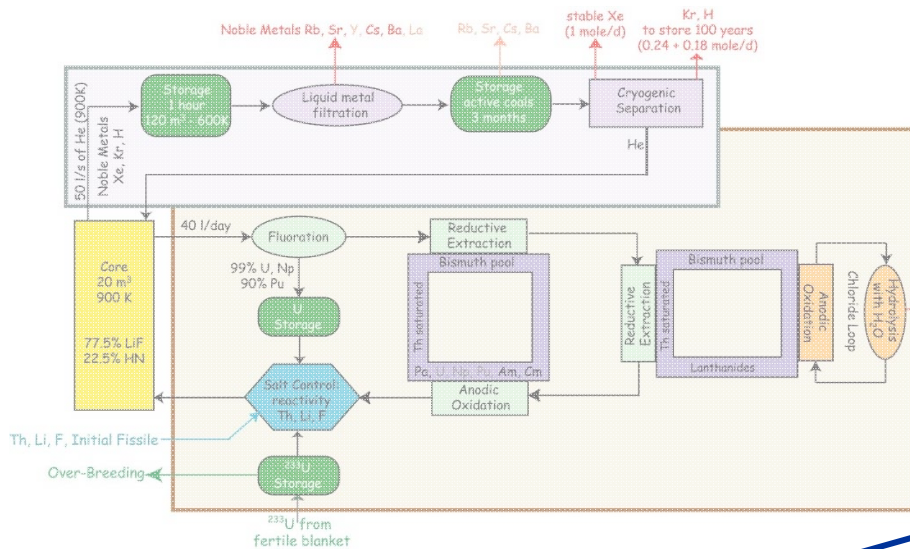
WP2 Physical and chemical properties required for safety analysis

WP3 Experimental proof of i) shut-down concept and ii) natural circulation dynamics for internally heated molten salt

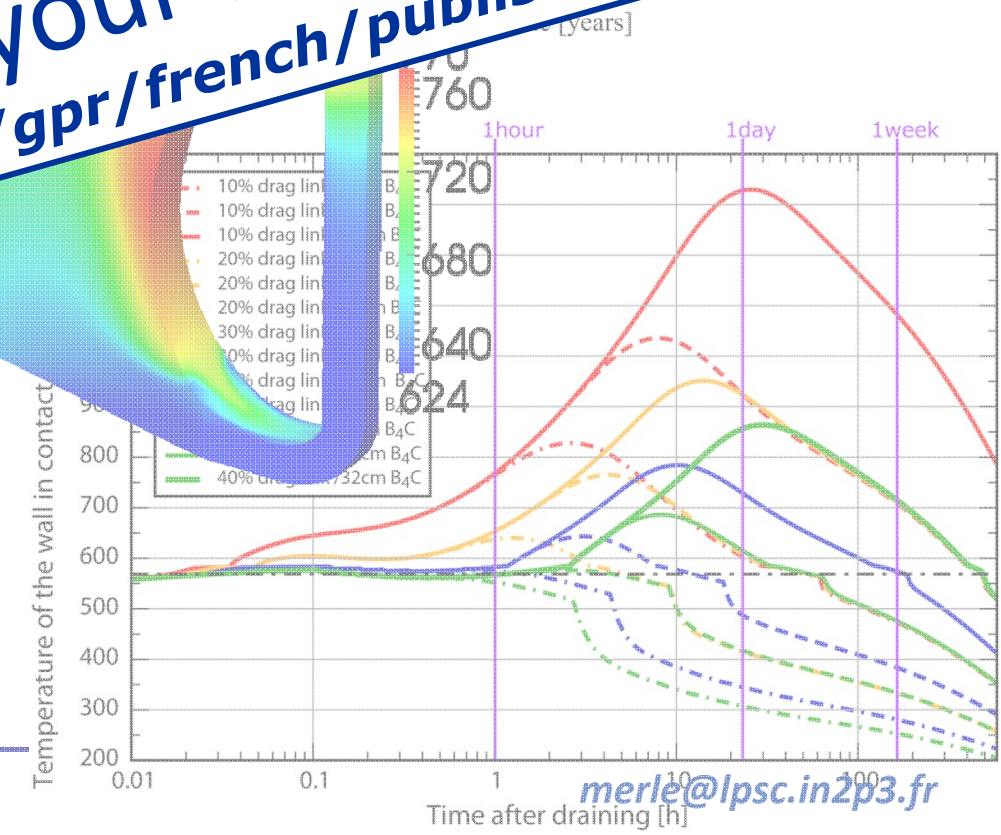
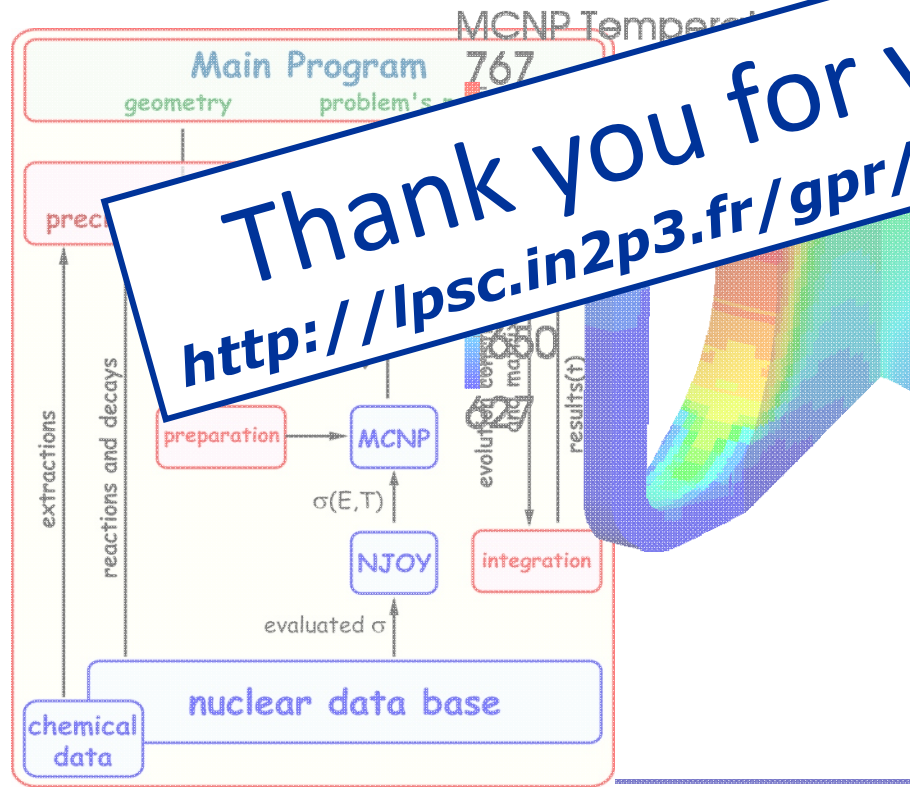
WP4 Accident analysis

WP5 Safety evaluation of the chemical plant





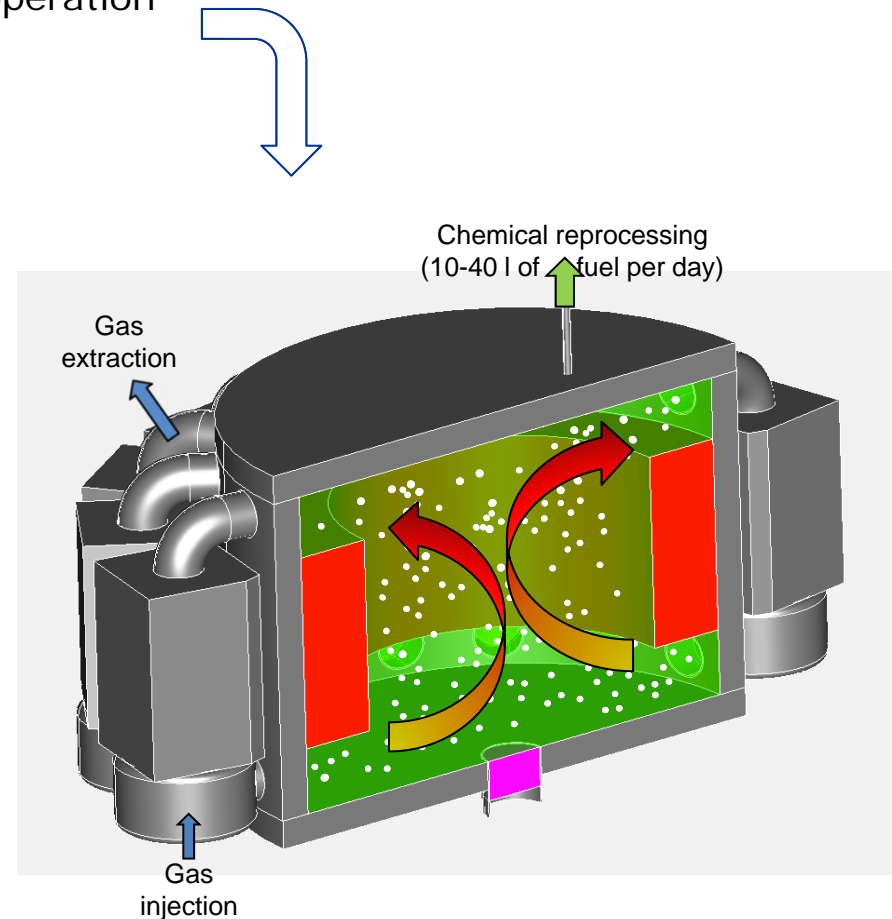
Thank you for your attention!
<http://ipsc.in2p3.fr/gpr/gpr/french/publis-rsf.htm>



MSFR: R&D collaborations

4th Generation reactors => Breeder reactors

Fuel reprocessing mandatory to recover the produced fissile matter – Liquid fuel = reprocessing during reactor operation



MSFR: R&D collaborations

4th Generation reactors => Breeder reactors

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Conclusions of the studies: **very low impact of the reprocessings (chemical and bubbling) on the neutronic behavior of the MSFR thanks to the fast neutron spectrum** = neutronic and chemical (physico-chemical properties of the salt) studies driven in parallel

PhD Thesis of X. Doligez

Studies requiring multidisciplinary expertise (reactor physics, simulation, chemistry, safety, materials, design...)



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