

# Neutronic-Thermohydraulic-Thermomechanic Coupling for the Modeling of Accidents in Nuclear Systems

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# Outline

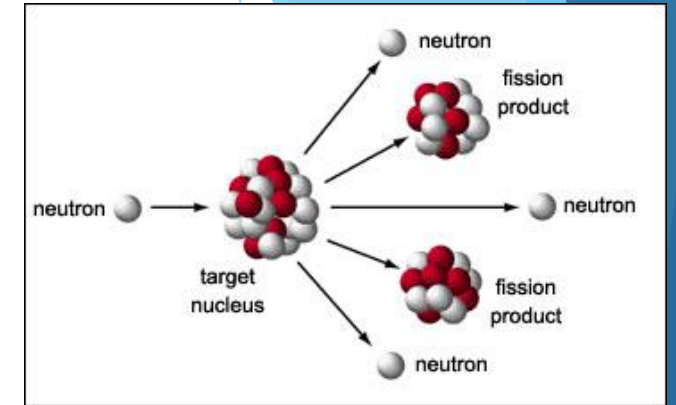
1. Criticality and Basics of Nuclear Physics
2. Thesis Subject
3. Multi-Physics Coupling
4. Results
5. Conclusions

# 1. Criticality Accidents

*A criticality accident is an involuntary and uncontrolled fission chain reaction*



The “Tickling the Tail of the Dragon” accident (Los Alamos, 1945) - Source Atomic Heritage Foundation



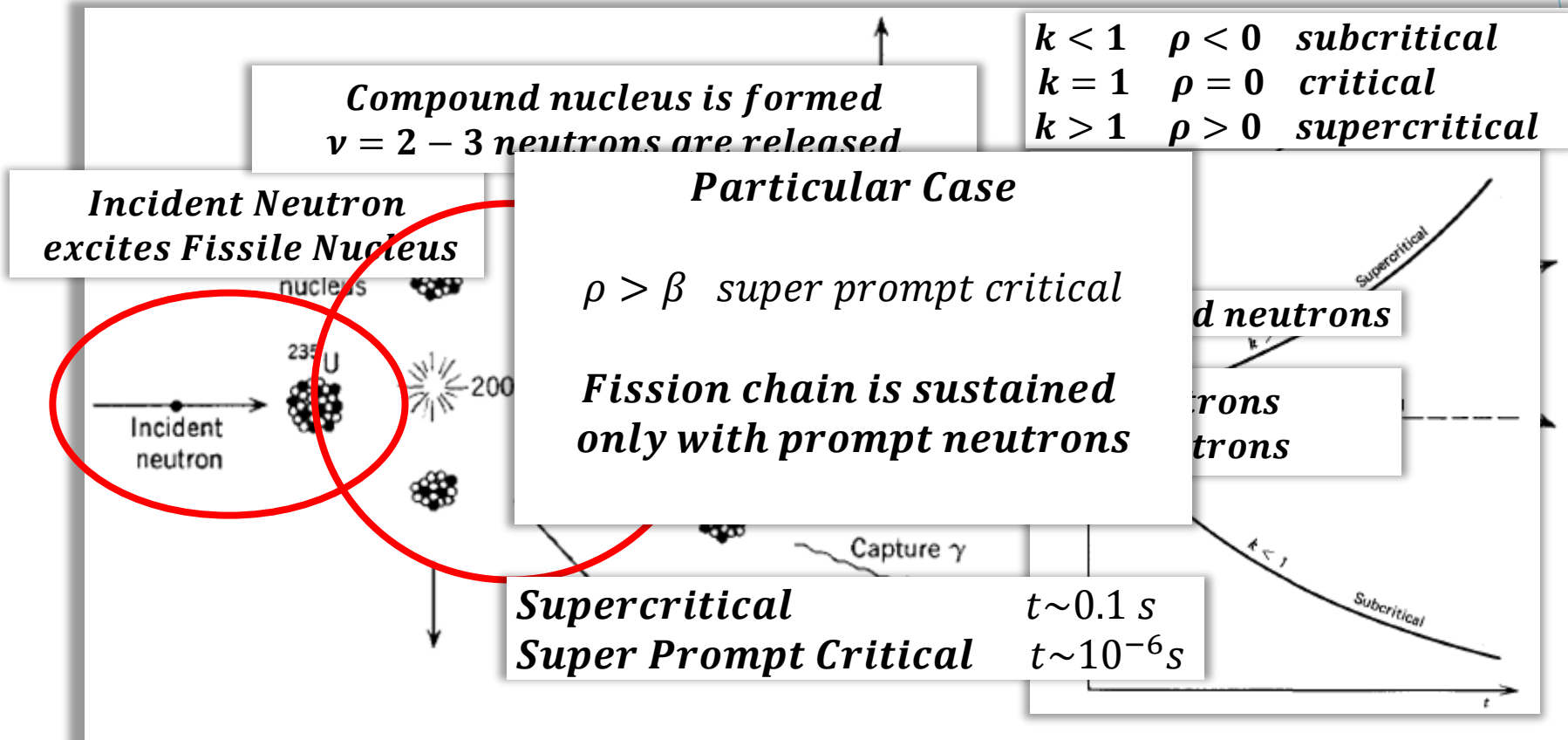
It can occur in nuclear systems involving very different

- **Geometric configurations**
- **Phases**
- **Phenomena**

Recent accidents: Tokai-Mura (Japan 1999, 3 deaths), etc.

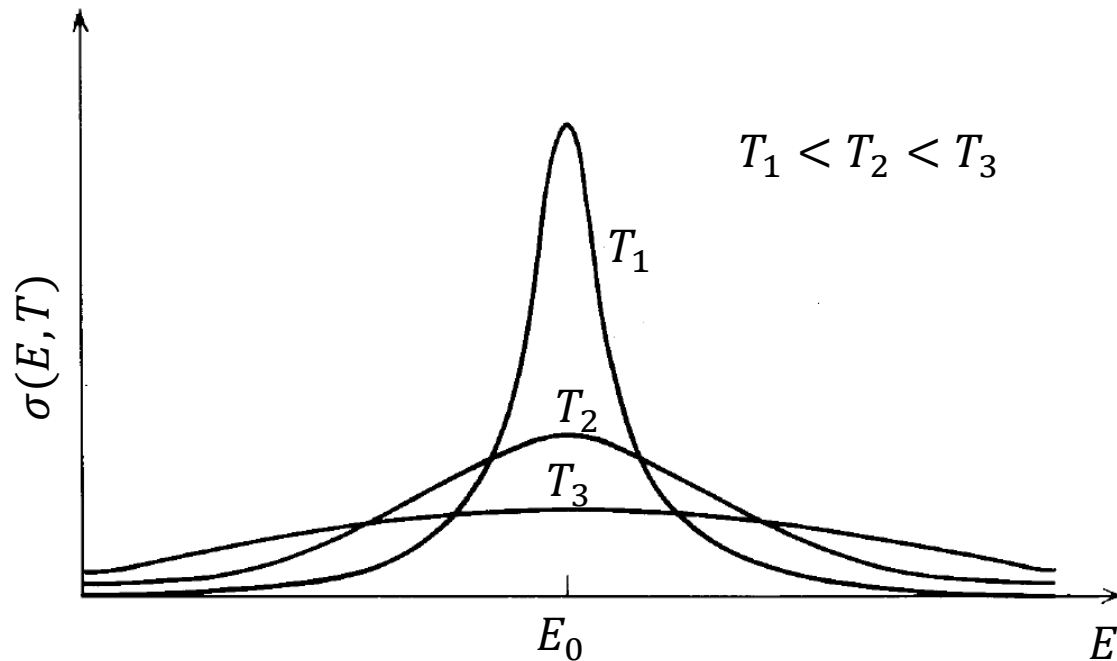
# 1. Nuclear Physics: Basic concepts

$$k_p \equiv \text{Prompt Multiplication Factor} = \frac{\text{Number of prompt neutrons in one generation}}{\text{Number of neutrons in preceding generation}} \equiv \text{Reactivity} \equiv \frac{k - 1}{k}$$



Fission Chain Reaction

# 1. Nuclear Physics: Basic concepts



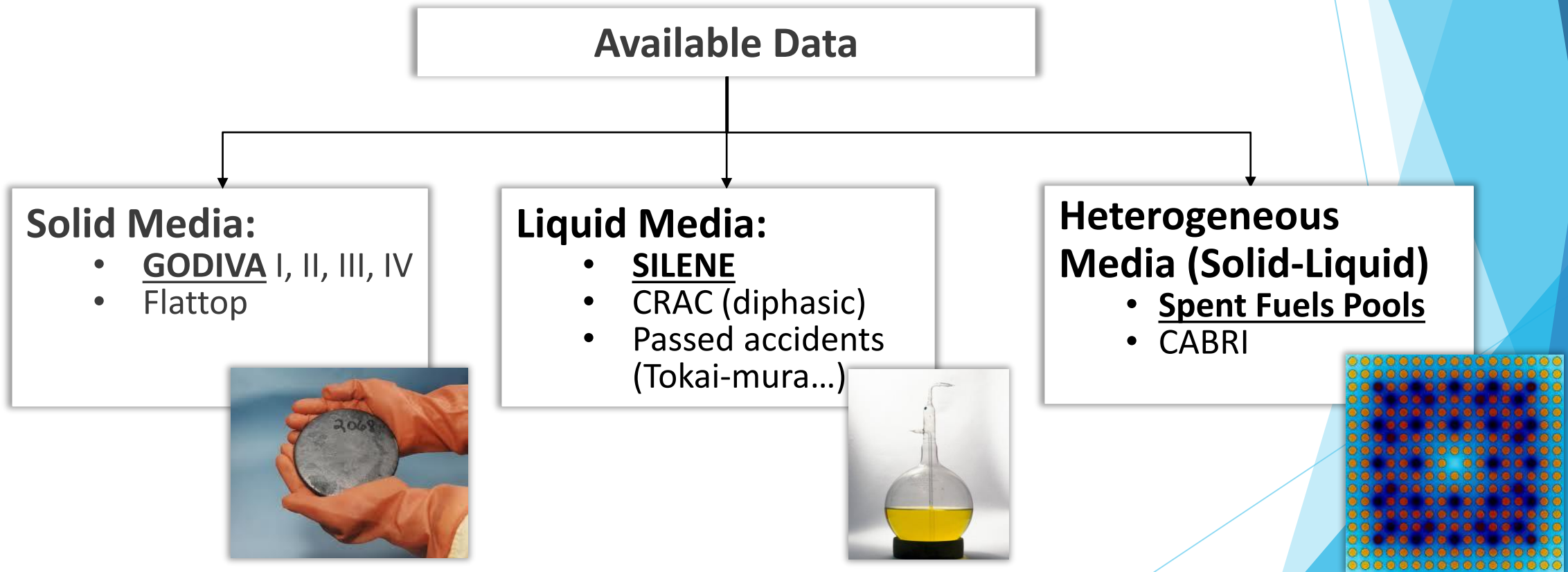
Doppler Broadening

- Dependence of neutron cross sections on the relative velocity between **neutron and nucleus**
- Target nuclei are in **continual motion** due to their **thermal energy**
- With increasing temperature the nuclei vibrate more rapidly within their lattice structures
- **Broadening** of the energy range of neutrons that may be resonantly absorbed in the fissile

Other Feedbacks exists like density change and geometry expansion (Leakage)

# 1. Criticality Accidents and Experiments

- Variety of accidents and experiments were reviewed
- Goal: select cases to cover a wide range of phenomena



# 2. Thesis Subject

## 2.1 State of Art

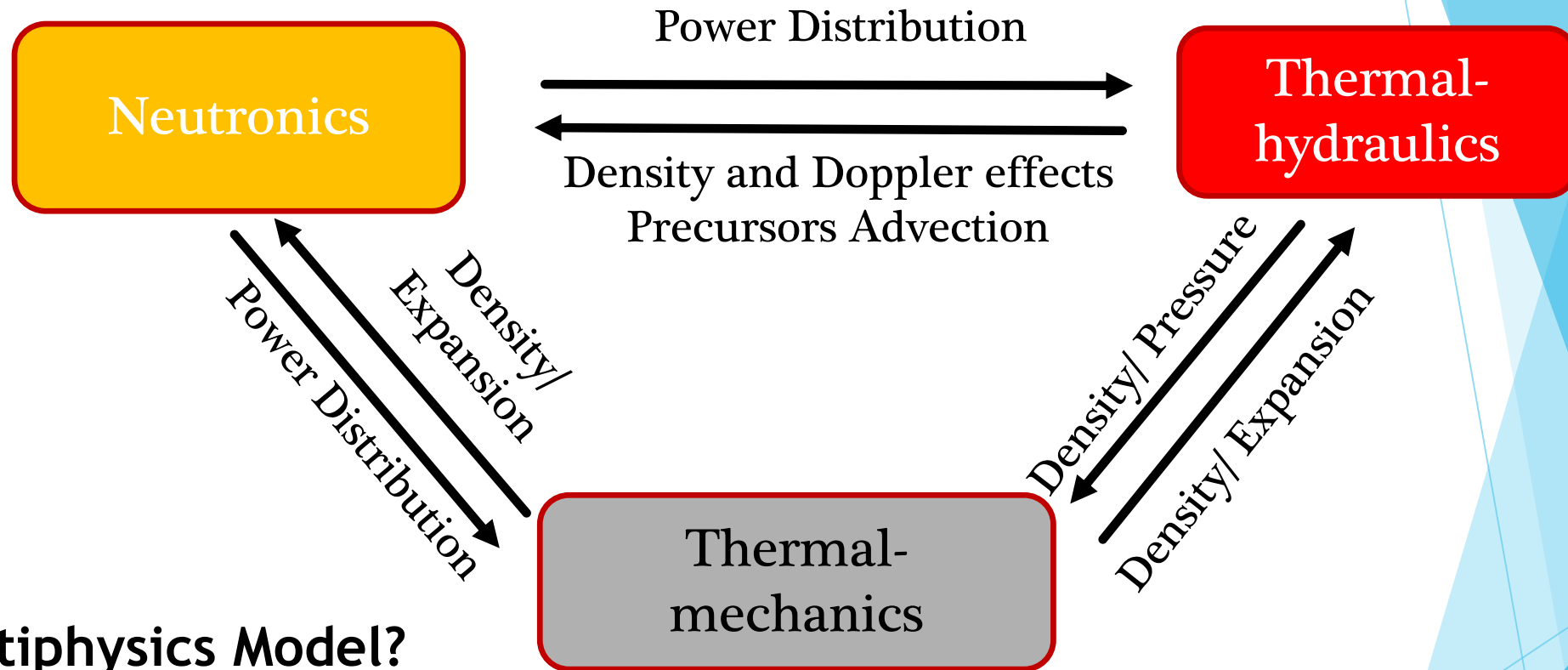
- ▶ **Current numerical models used by the safety authority limited for criticality accidents in:**
  - ▶ Geometry modelling
  - ▶ Transient simulated time



## 2.2 Objective

- ▶ **Develop a more general transient multi-physics multiscale tool with:**
  - ▶ Detailed phenomena modelling
  - ▶ Higher space/time scale flexibility
  - ▶ Best-estimate (Not conservative)

### 3. Transient Multi-physics Multiscale Tool



#### Why Multiphysics Model?

- ▶ Mechanistic model
- ▶ Account for all relevant phenomena
- ▶ High time/space scale flexibility



### 3. Multi-physics Tool: the Bricks/Codes

CFD C++ Library

Open  FOAM

- ▶ **OpenFOAM** is an open source software based in C++ for numerical resolution of the continuum mechanics including CFD

Monte Carlo Code (Serpent)



- ▶ **Serpent 2** is a 3D continue in energy Monte Carlo code for reactor physics and irradiation calculus (burnup)

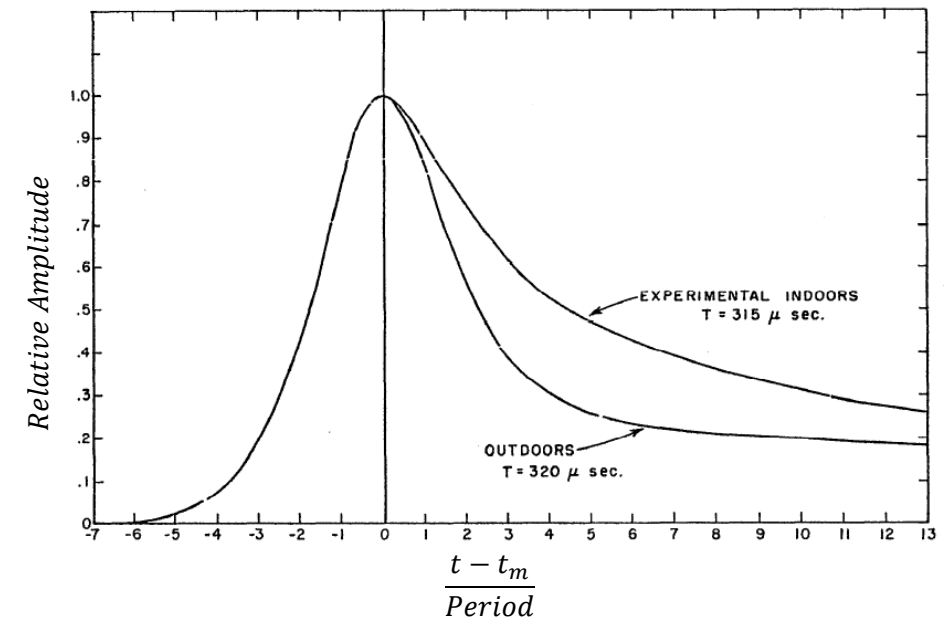
# Multi-physics Coupling

- Godiva Experiment
- Neutronics
- Thermomechanics

# 3. Multi-physics Coupling

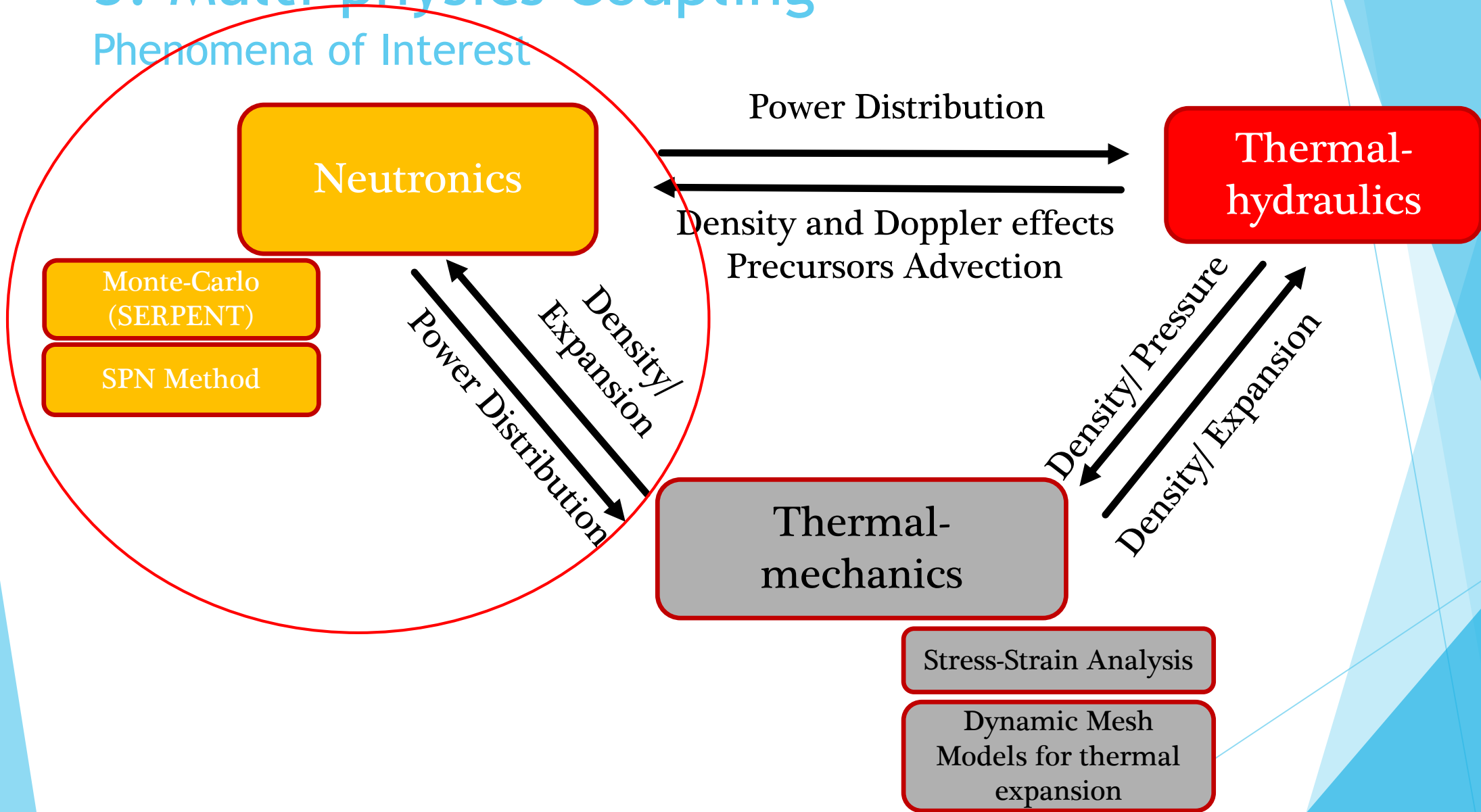
## Godiva Experiment

- ▶ Experiment description:
  - ▶ **Geometry:** sphere
  - ▶ **Size:** ~8.85 cm radius
  - ▶ **Fuel:** enriched Uranium (95%)
  - ▶ **Mass:** ~54 kg
  - ▶ **Reactivity control mechanisms:** none
    - ⇒ only neutronics feedback effects
- ▶ Key phenomena to be modeled:
  - ▶ Super prompt critical transient ( $\rho > \beta$ )
  - ▶ Thermal expansion (density and leakage feedback)
  - ▶ Doppler effect (temperature feedback )



# 3. Multi-physics Coupling

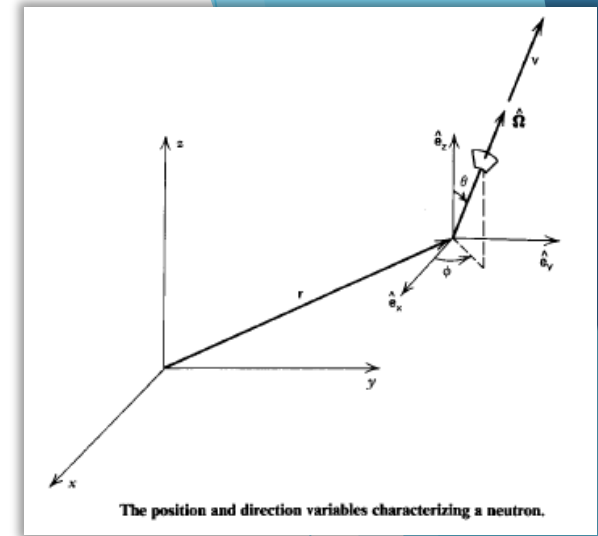
Phenomena of Interest



# 3. Multi-physics Coupling

## Neutronics

- ▶ Neutron population described by Boltzmann equation and a balance of precursors with an advection term is used (in case of liquid fuels)



$$\underbrace{\frac{1}{v(E)} \frac{\partial \psi}{\partial t}(\vec{r}, \vec{\Omega}, E, t)}_{\text{Rate of change}} = \underbrace{\left[ -\mathcal{L} - \mathcal{T} + \mathcal{S} + \frac{\chi_p(E)}{4\pi} (1 - \beta) F \right]}_{\text{Streaming + Disappearance + Scattering + Fissions}} \psi(\vec{r}, \vec{\Omega}, E, t) + \underbrace{\sum_{d=1}^{G_d} \frac{\chi_d(E)}{4\pi} \lambda_d C_d(\vec{r}, t)}_{\text{Delayed Neutrons source}}$$

Rate of change

Streaming + Disappearance + Scattering + Fissions

Delayed  
Neutrons source

$$\underbrace{\frac{\partial C_d}{\partial t}(\vec{r}, t)}_{\text{Local rate of change}} = \underbrace{\beta_d F \psi(\vec{r}, \vec{\Omega}, E, t)}_{\text{Production}} - \underbrace{\lambda_d C_d(\vec{r}, t)}_{\text{Destruction}} - \underbrace{\vec{u} \cdot \nabla C_d(\vec{r}, t)}_{\text{Convection}} + \underbrace{D_d \nabla^2 C_d(\vec{r}, t)}_{\text{Diffusion}} \quad \text{for } d = 1 \text{ to } G_d$$

Local rate  
of change

Production

Destruction

Convection

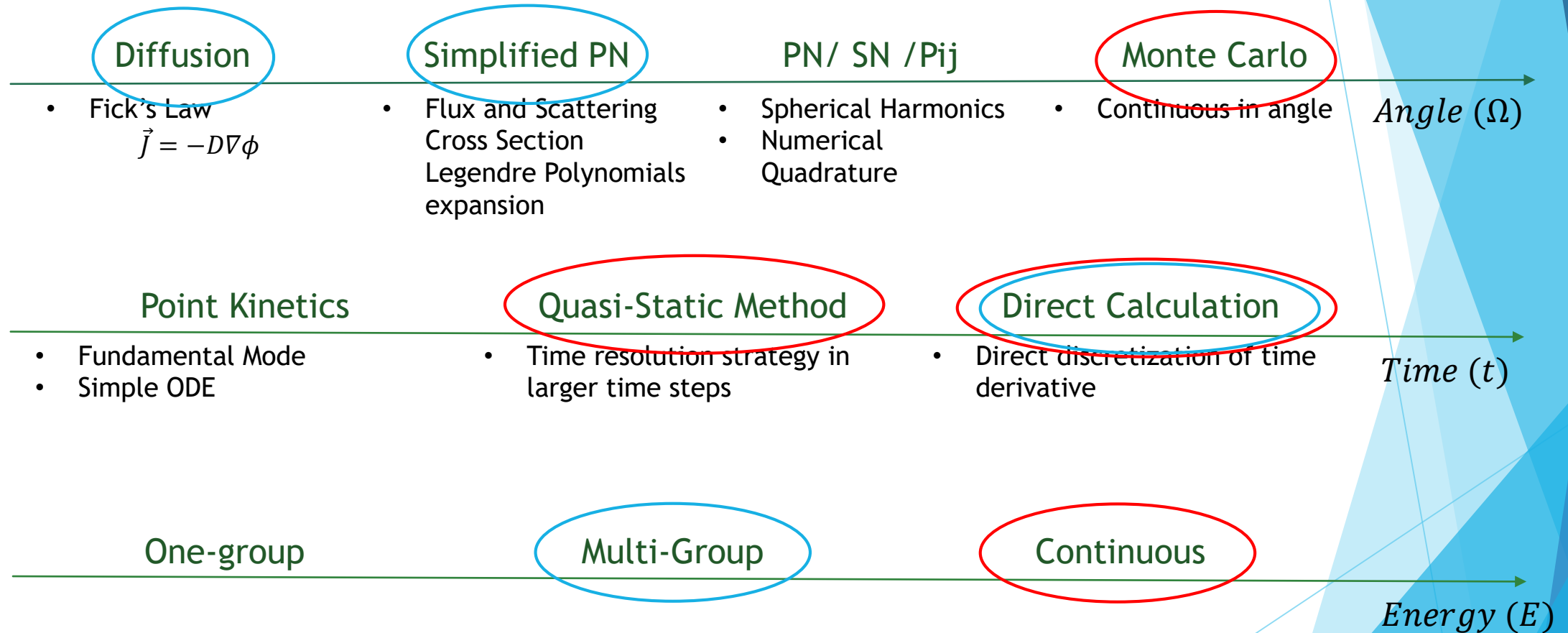
Diffusion

Liquid Media

Neutronics

# 3. Multi-physics Coupling

## Neutronics methods and strategy



# 3. Multi-physics Coupling

## Neutronics: A) the Simplified PN

- ▶ The **transient multigroup SP3** equations consist in a set of two coupled PDEs
- ▶ The **order 0** is identical to diffusion approximation equation
- ▶ The **order 2** takes into account anisotropies in the scattering cross section with a **Legendre Polynomial Expansion**

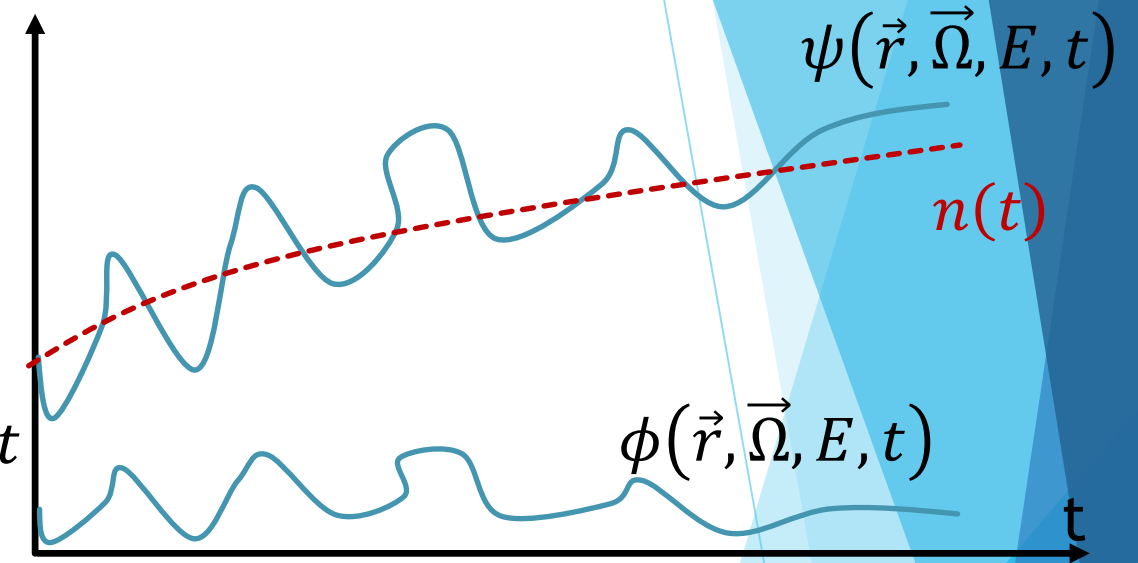
$$\left\{ \begin{array}{l} \frac{1}{V} \frac{\partial \hat{\phi}_0}{\partial t} = \nabla \frac{1}{3} \Sigma_1^{-1} \nabla \hat{\phi}_0 - \Sigma_0 \hat{\phi}_0 + \frac{F}{k} (\hat{\phi}_0 - 2\phi_2) + 2\Sigma_0 \phi_2 + \frac{2}{V} \frac{\partial \phi_2}{\partial t} + S_d \quad \text{order 0} \\ \frac{3}{V} \frac{\partial \phi_2}{\partial t} = \nabla \frac{3}{7} \Sigma_3^{-1} \nabla \phi_2 - \left( \frac{5}{3} \Sigma_2 + \frac{4}{3} \Sigma_0 \right) \phi_2 - \frac{2}{3} \frac{F}{k} (\hat{\phi}_0 - 2\phi_2) + \frac{2}{3} \Sigma_0 \hat{\phi}_0 + \frac{2}{3V} \frac{\partial \hat{\phi}_0}{\partial t} - \frac{2}{3} S_d \quad \text{order 2} \end{array} \right.$$

# 3. Multi-physics Coupling

## Neutronics: B) the Quasi-Static Method

### Key hypothesis:

$$\left\{ \begin{array}{l} \psi(\vec{r}, \vec{\Omega}, E, t) = n(t) \phi(\vec{r}, \vec{\Omega}, E, t) \\ \left\langle \frac{1}{V(E)} \phi(\vec{r}, \vec{\Omega}, E, t) \mid W_0(\vec{r}, \vec{\Omega}, E) \right\rangle = constant \end{array} \right.$$



- ▶ First hypothesis: separation of the neutron angular flux into an amplitude function  $n(t)$  and a shape function  $\phi(\vec{r}, \vec{\Omega}, E, t)$
- ▶ Second hypothesis: makes the two separated functions unique



# 3. Multi-physics Coupling

## Neutronics: the Quasi-Static Method

### Transport Equations

$$\frac{1}{v} \frac{\partial \psi}{\partial t} = \mathcal{L}\psi + \sum_{d=1}^{G_d} \frac{\chi_d}{4\pi} \lambda_d C_d$$

$$\frac{\partial C_d}{\partial t} = \beta_d F \psi - \lambda_d C_d$$

The QS method allows splitting the neutron transport equation in two sets of equations:

- Neutron flux **shape (PDE)** and
- Neutron flux **amplitude (ODE)**

Neutron flux shape (PDE)  $\frac{\partial \phi}{\partial t} = 0$  Neutron flux amplitude (ODE)  $\frac{dn}{dt} = 0$  Improved Quasi Static

$$\frac{1}{v} \frac{\partial \phi}{\partial t} = \left[ \mathcal{L} - \frac{1}{v} \frac{dn(t)/dt}{n(t)} \right] \phi + \frac{1}{n(t)} \sum_{d=1}^{G_d} \frac{\chi_d}{4\pi} \lambda_d C_d$$

$$\frac{\partial C_d}{\partial t} = \beta_d F \phi n(t) - \lambda_d C_d$$

### Neutron flux Amplitude Equations

$$\frac{dn(t)}{dt} = \left[ \frac{\rho - \beta_d^{eff}}{\Lambda} \right] n(t) + \sum_{d=1}^{G_d} \lambda_d \bar{c}_d(t)$$

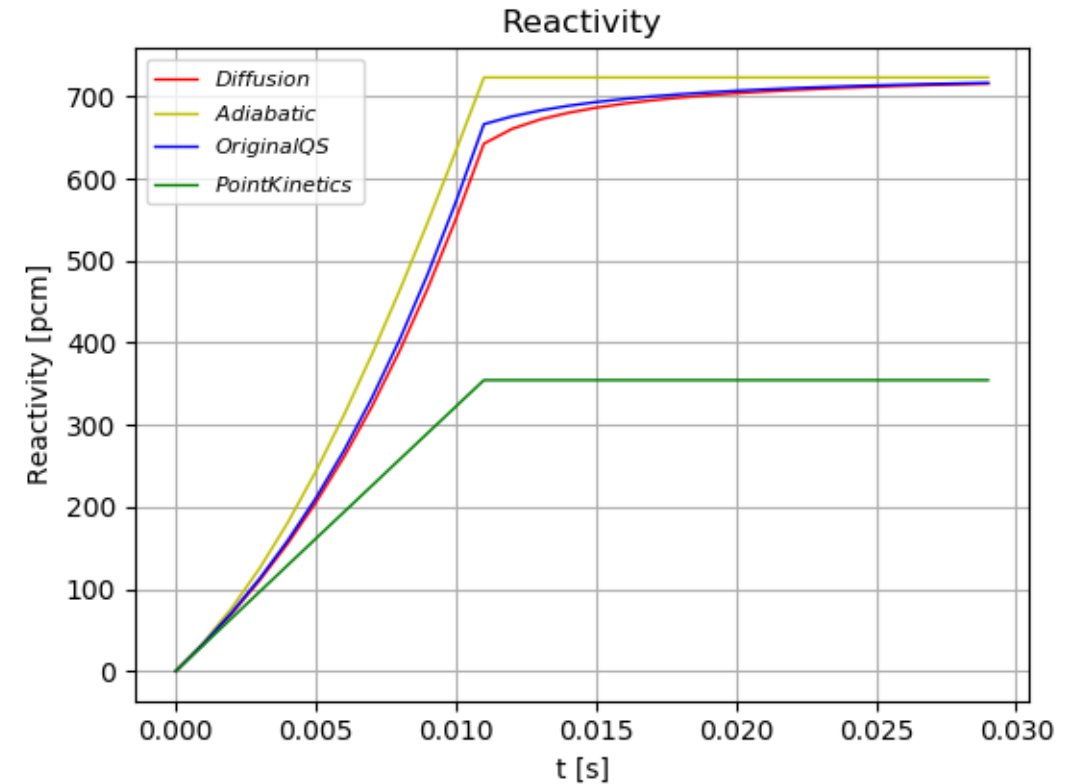
$$\frac{d\bar{c}_d(t)}{dt} = \frac{\beta_d^{eff}}{\Lambda} n(t) - \lambda_d \bar{c}_d(t)$$

# 3. Multi-physics Coupling

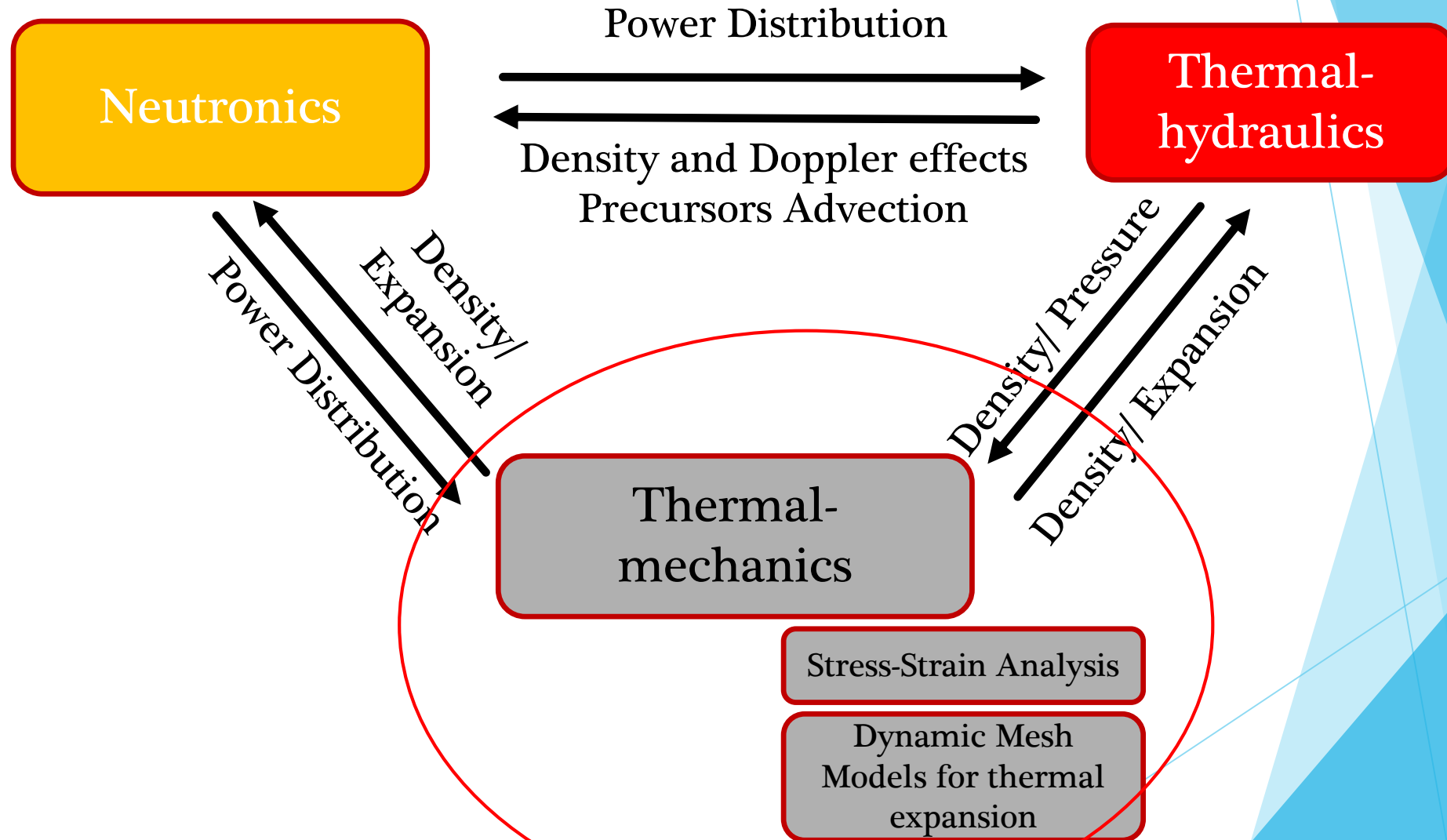
## Neutronics: the Quasi-Static Method variants

$$\left\{ \begin{array}{ll} \frac{\partial \phi}{\partial t} \neq 0 & \frac{dn(t)}{dt} \neq 0 \quad \text{Improved Quasi Static} \\ \frac{\partial \phi}{\partial t} = 0 & \frac{dn(t)}{dt} \neq 0 \quad \text{Original Quasi Static} \\ \frac{\partial \phi}{\partial t} = 0 & \frac{dn(t)}{dt} = 0 \quad \text{Adiabatic Quasi Static} \end{array} \right.$$

- ▶ Sensibility study for 90\$/\$s (p/B) reactivity increase was made using diffusion theory
- ▶ Adiabatic case (yellow) seems to be the less accurate but it is still better than point kinetics (green) alone
- ▶ Adiabatic case will be used for Monte Carlo calculations



### 3. Multi-physics Coupling



# 3. Multi-physics Coupling

## Thermal-Mechanics Model

- ▶ A linear elastic solid model with thermal expansion was used to calculate the displacement field  $D$
- ▶ The governing equation are obtained from the force balance for the solid body element

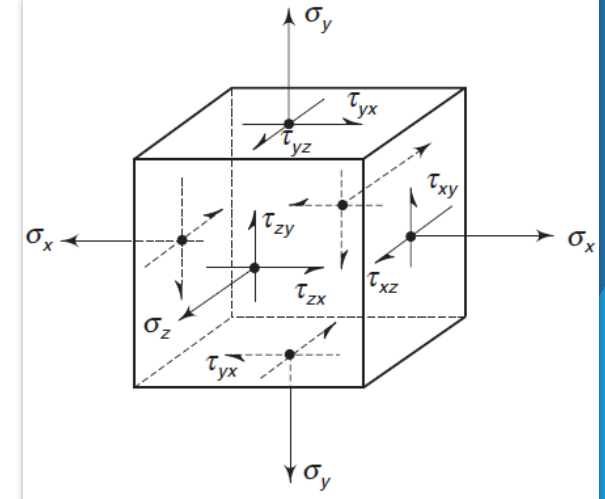
$$\frac{\partial^2(\rho D)}{\partial t^2} = \nabla[\mu \nabla D + \mu (\nabla D)^T + \lambda \text{tr}(\nabla D)] + \nabla \left( \frac{E}{1-2\nu} \alpha T \right)$$

- ▶ The temperature field  $T$  is calculated via the heat transfer equation

$$\frac{\partial(\rho c T)}{\partial t} = \nabla(k \nabla T) + \underbrace{q'''_{fission}}_{\text{Coupling term}}$$

- ▶ Important for thermal expansion and density feedback

$$\sigma = 2\mu\epsilon + \lambda \text{tr}(\epsilon)I$$
$$\epsilon = \frac{1}{2}(\nabla D + \nabla D^T)$$

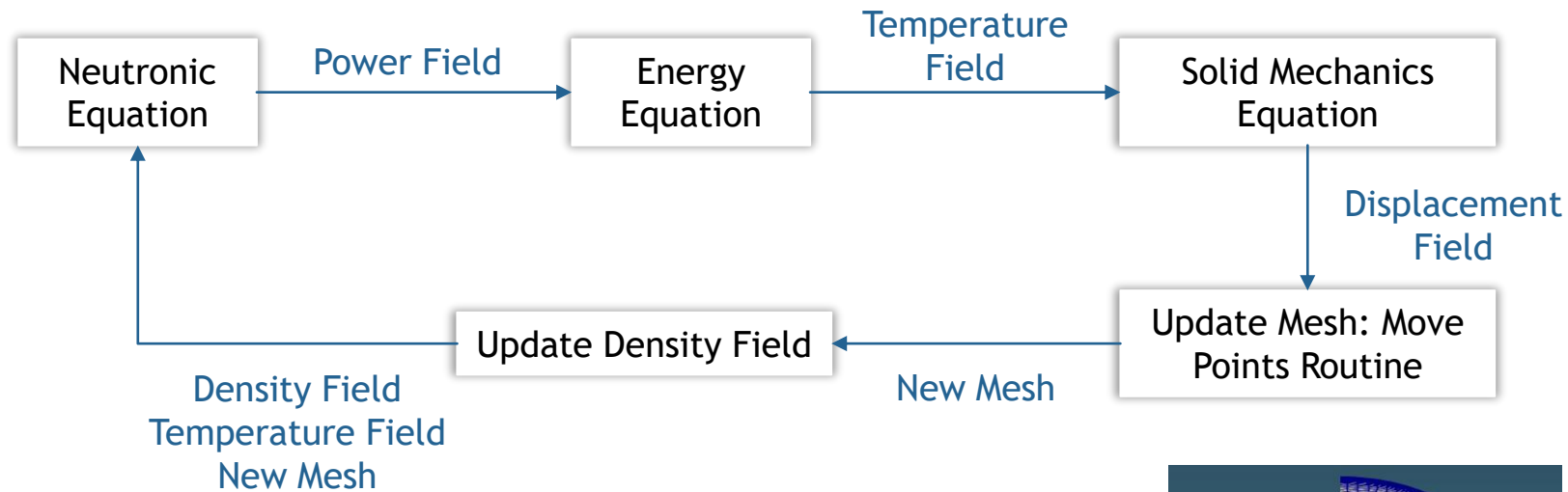


Differential Element Force Balance

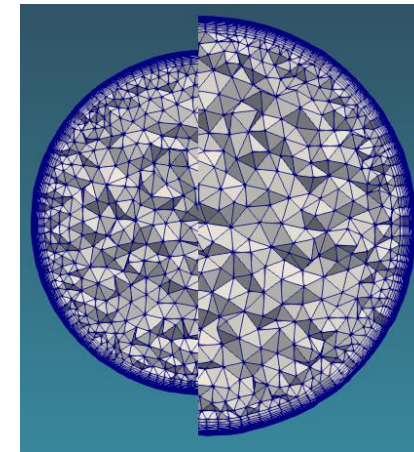
Thermal-  
mechanics

# 3. Multi-physics Coupling

## Implementation of the example for the Godiva Experiment



- Mesh discretization (~100000 cells)
- Adaptive mesh for thermal expansion implemented in OpenFOAM
- Density fields updated for accounting geometry changes



# Results

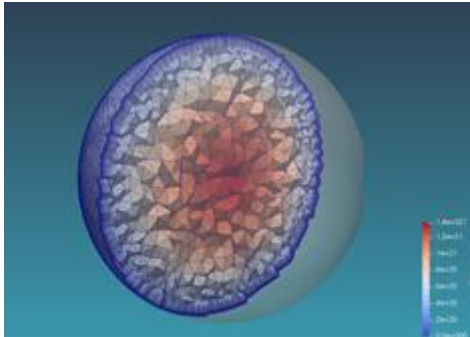
- Monte Carlo Quasi Static
- SPN

# 4. Results

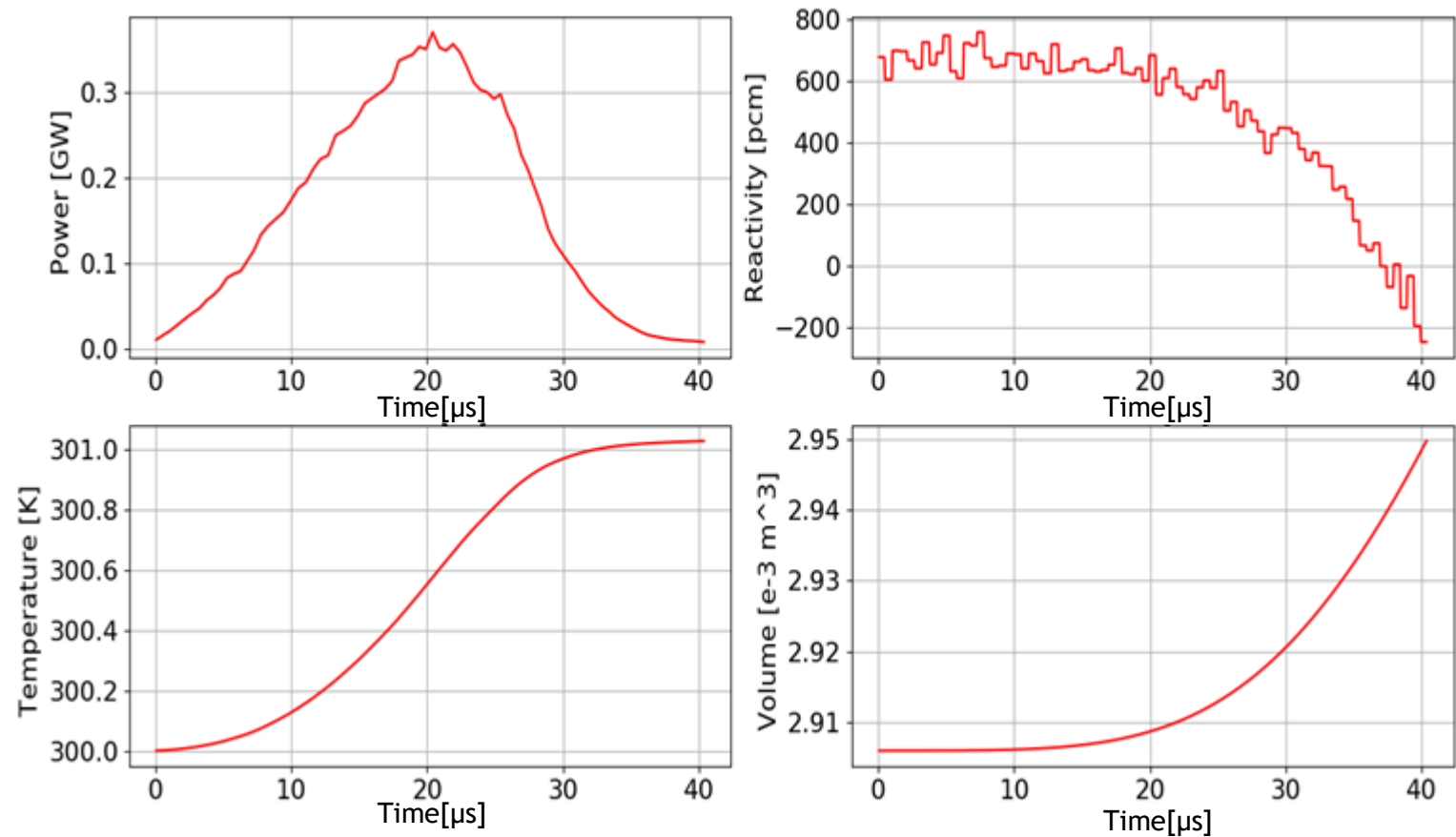
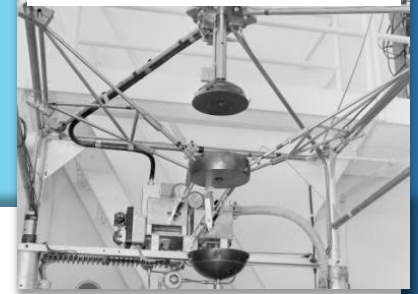
## Serpent Quasi-Static Stochastic Approach

- ▶  $\frac{\rho}{\beta} \sim 1.06 \$$
- ▶  $r \sim 8.85 \text{ cm}$
- ▶  $ExecutionTime = 4.1h$
- ▶ 1 processor 1.7Ghz  
(*OpenFOAM*)
- ▶ 10 processors 1.7Ghz  
(*Serpent*)

Flux



Godiva Experiment

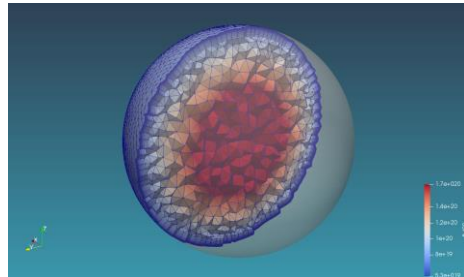


# 4. Results

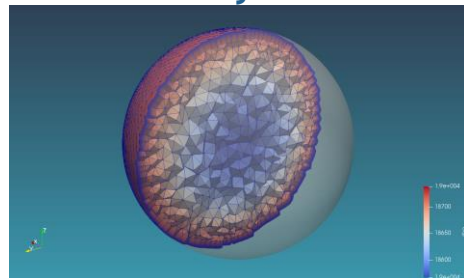
## SPN Deterministic Approach

- ▶  $\frac{\rho}{\beta} \sim 1.015 \$$
- ▶  $r \sim 8.4 \text{ cm}$
- ▶ *ExecutionTime* = 2.1h
- ▶ 1 processor 1.7Ghz

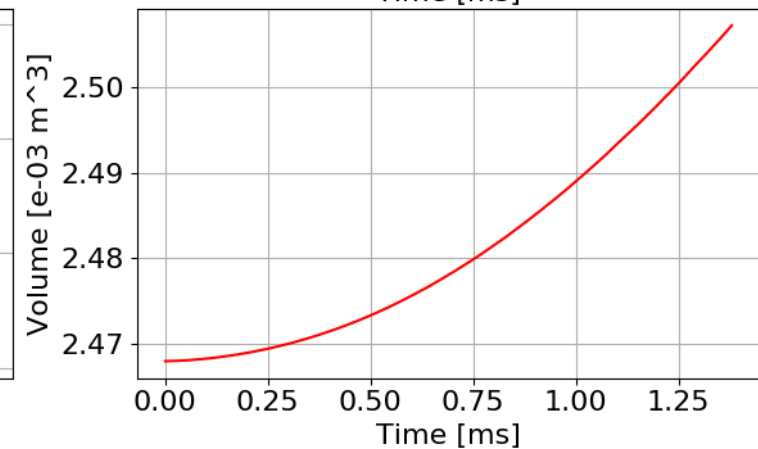
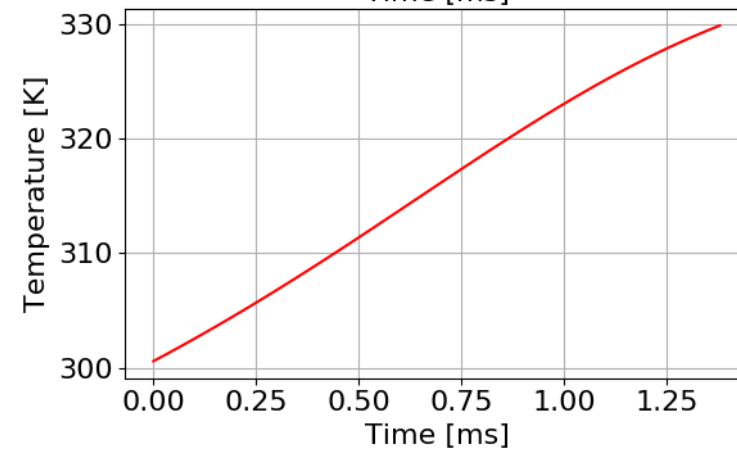
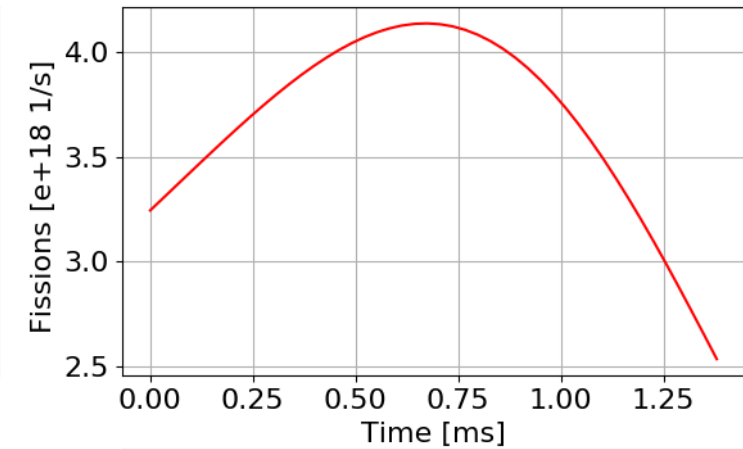
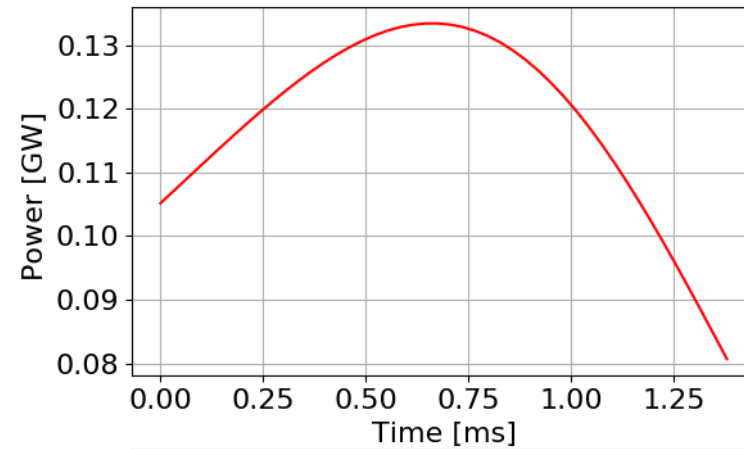
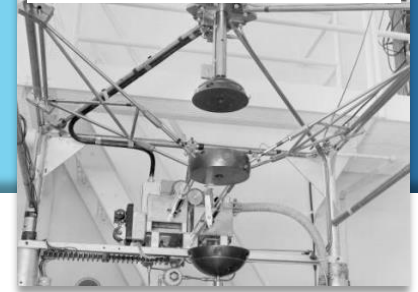
Flux Field Order 0 Energy Group 1/8



Density Field



Godiva Experiment





# 4. Results

## Discussion for the Godiva Experiment

- ▶ Both SP3 and Serpent QS provide consistent simulation results to experimental data
  - ▶ The initial reactivity ( $k_{\text{eff}}$ ) obtained by the SP3 and Serpent methods were not the same in these preliminary results due to the approximations made by each method
  - ▶ A more precisely evaluation is currently underway to obtain closer initial conditions
- ▶ Calculation Time: SP3 is quicker than Quasi-Static serpent but the latter is more precise
- ▶ Advantage SP3: useful for quicker testing of other parts of the coupling (TM or TH)
- ▶ Cross Section data for SP3: condensed in Serpent taking into account Legendre polynomial expansion. This step is time consuming and has to be added to the total calculation time

# Conclusions

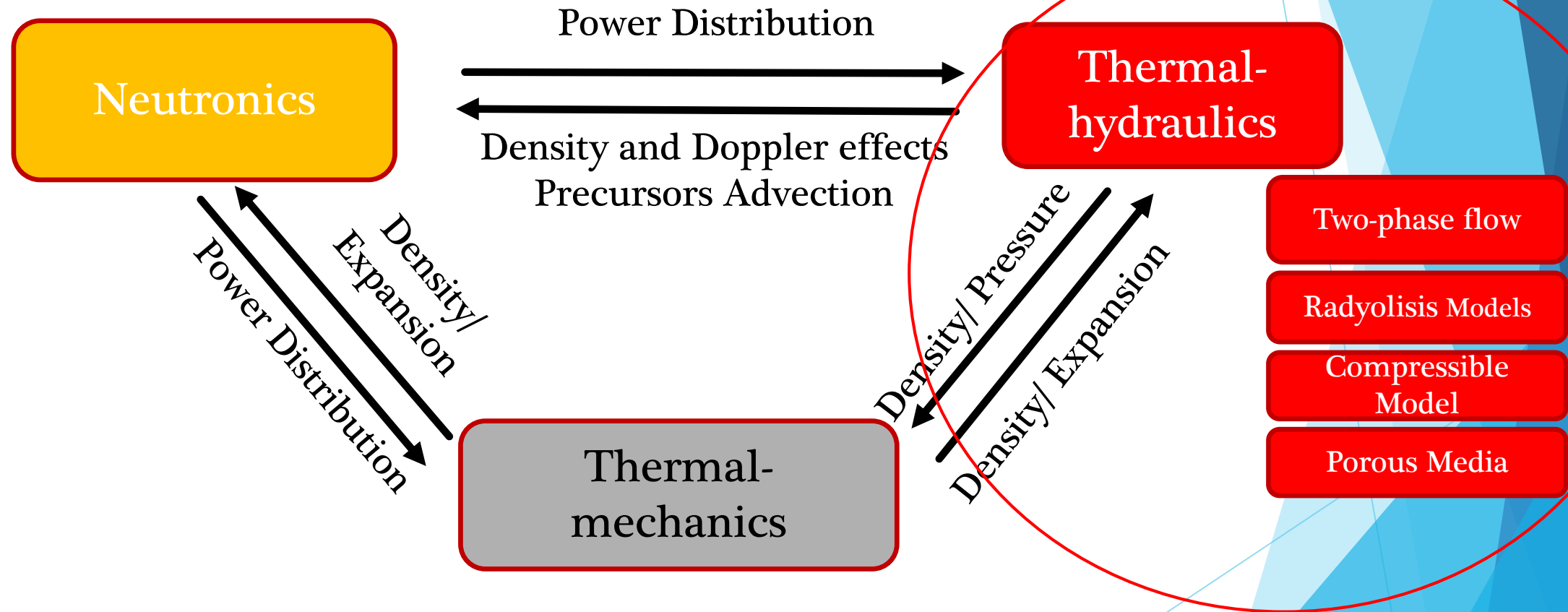
- Godiva
- On-going Work

# 5. Conclusions

- ▶ Good agreement for Godiva transient was obtained
- ▶ The adiabatic method is inaccurate for extreme transients (90\$/s). Still it is a better estimation than point kinetics alone
- ▶ Three neutronics methods have been implemented in the multiphysics tool allowing covering:
  - ▶ Larger spectrum of sizes, times and energy

# 5. Conclusions

On-going Work: SILENE



# 5. Conclusions

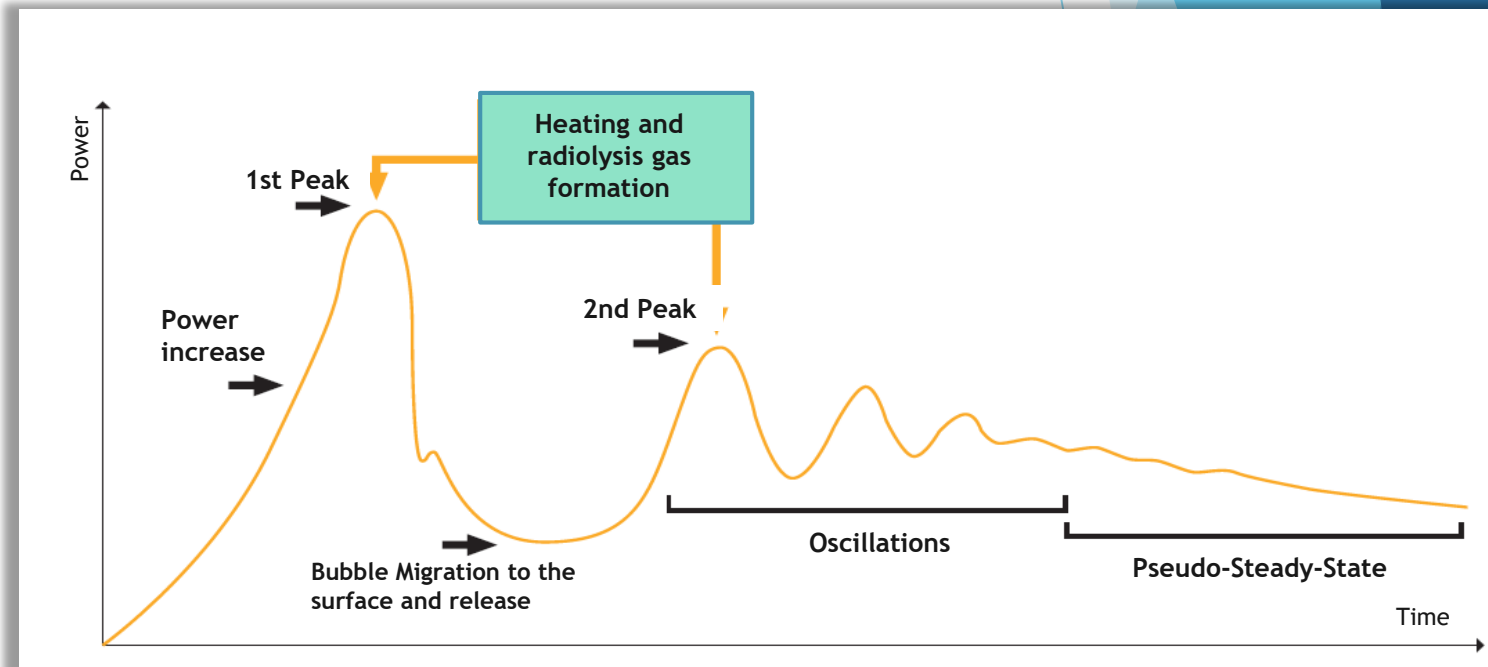
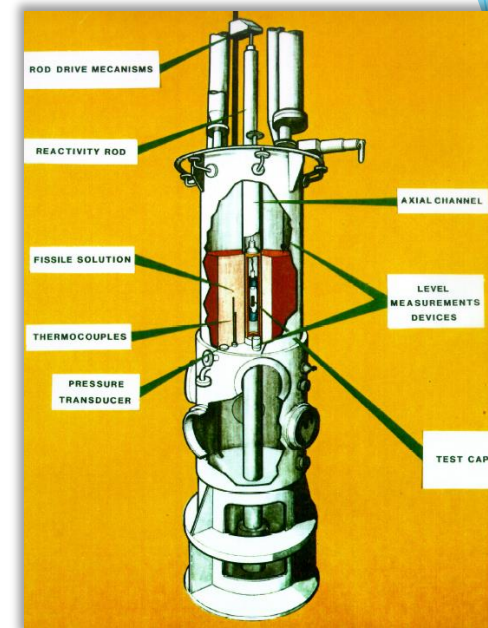
## On-going Work: Liquid Media -> SILENE

### ▶ Experiment description:

- ▶ **Geometry:** Annular cylinder
- ▶ **Size:** 36 cm diameter and ~23 cm height
- ▶ **Fuel:** solution of enriched uranyl nitrate (~93%)
- ▶ **Reactivity control mechanisms:**
  - ▶ Control rod
  - ▶ Liquid fuel level

### ▶ Principal Phenomena

- ▶ Super prompt critical transient ( $\rho > \beta$ )
- ▶ Precursors transport
- ▶ Radiolysis: gas phase production
- ▶ Pressure waves
- ▶ Free surface sloshing

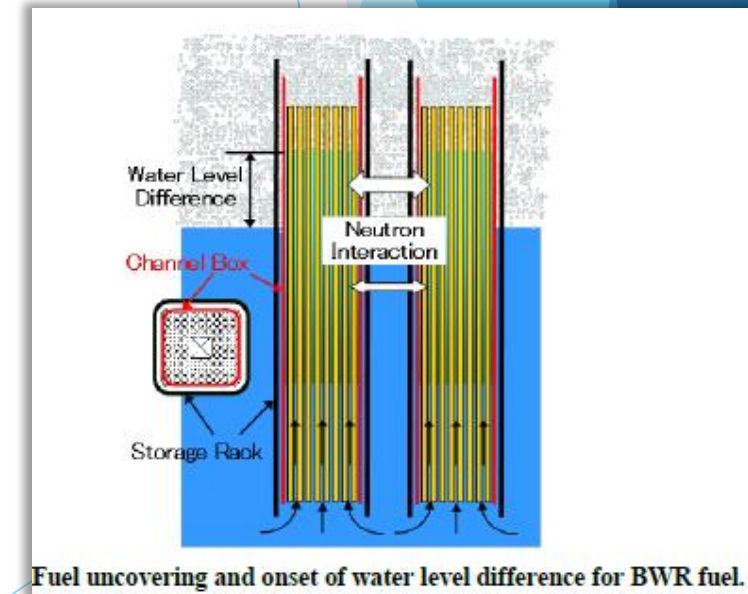
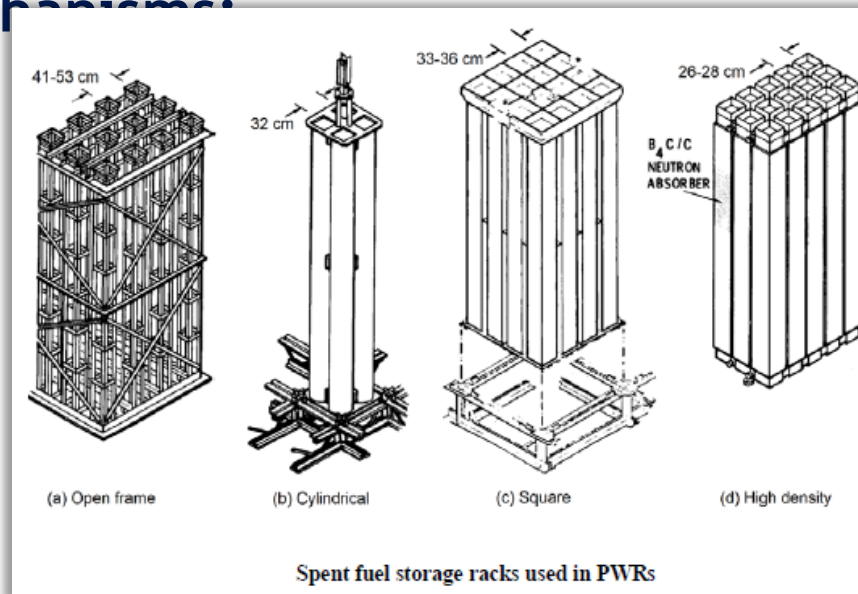
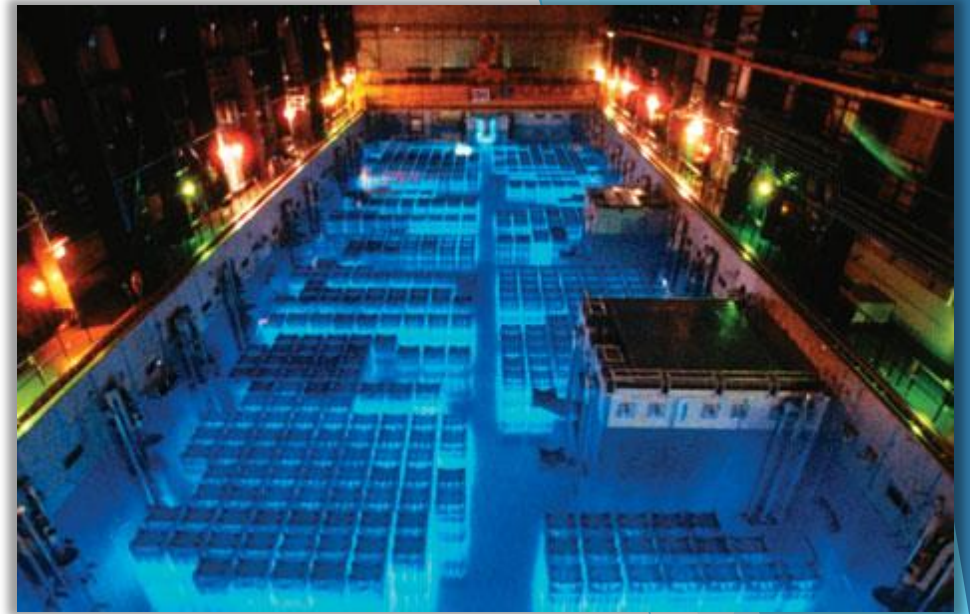


# 5. Conclusions

On-going Work:

Heterogeneous Media -> Spent Fuel Pools

- ▶ Experiment description:
  - ▶ **Geometry:** Assemblies grouped in racks
  - ▶ **Fuel:** PWR/BWR Assemblies
  - ▶ **Reactivity control mechanisms:**
    - ▶ Neutron Poisons
- ▶ **Principal Phenomena**
  - ▶ Biphase Porous Media
  - ▶ Criticality Margins



Fuel uncovering and onset of water level difference for BWR fuel.

# Thank you